

LICENSEE EVENT REPORT

CONTROL BLOCK:

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1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7 8 9 14 15 25 26 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

0 1 T N S N P 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5

LICENSEE CODE LICENSE NUMBER LICENSE TYPE JO CAT SV

CON'T

REPORT SOURCE: 01 L 6 0 5 0 0 0 3 2 7 7 1 0 2 8 8 3 8 1 1 2 5 8 3 9
60 61 DICTIONARY NUMBER 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 Unit 1 in mode 1 at 100% RTP. At 1620 (C) on 10/28/83, containment isolation valve

03 | 1-FCV-77-20 in the reactor coolant drain tank nitrogen supply line was found

0 4 | inoperable during surveillance testing. This event required entry into LCO 3.6.3.

05 There was no effect upon public health or safety. Previous occurrences - none.

06

07

0	8	
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0 9 8

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

9 10 11 12 13 14 15 16

C J 11 E 12 A 13 V A L V O P 14 D 15 Z 16

(17) LER/RO REPORT NUMBER [8 | 3]
 21 22

SEQUENTIAL REPORT NO. [1 | 5 | 2]
 24 25 26

OCCURRENCE CODE [0 | 3]
 28 29

REPORT TYPE [L]
 30

REVISION NO. [—]
 31

[—]
 32

ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS				ATTACHMENT SUBMITTED		NPD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURE		
X	18	A	19	Z	20	Z	21	0	0	0	0	N	23	N	24	N	25	I	2	0
33		34		35		36		37			40	41		42		43		44		

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The valve was isolated upon discovery. Investigation revealed a bad cable had

11 caused the valve inoperability. The cable will be replaced as soon as material

1 2 is available and proper work procedures are complete.

1	3	
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1 4

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
1	5	E	28	1	0	0	29	NA	
8	9	10	12	13	44	45	46	B	31
								Surveillance test	

ACTIVITY CONTENT
RELEASED OF RELEASE

1 6 2 33 2 34

8 9 10 11

AMOUNT OF ACTIVITY 35

NA

44

LOCATION OF RELEASE 36

NA

45

PERSONNEL EXPOSURES									
NUMBER			TYPE		DESCRIPTION				
1	7		0	0	0	17	Z	38	NA

PERSONNEL INJURIES	
NUMBER	DESCRIPTION
000	NA

9 11 12
LOSS OF OR DAMAGE TO FACILITY
TYPE DESCRIPTION (43)
[9] [Z] (42) NA
S312080074 S31125
PDR ADOCK 05000327
S PDR
IE22

9 10
PUBLICITY
ISSUED DESCRIPTION (45)
[] 0 [N] (44) NA
NRC USE ONLY

NRC USE ONLY v

Name of Preparer: G. B. Kirk /M. R. Harding

Phone: (615) 870-6422

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower 11

November 25, 1983

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW.
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET
NO. 50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE
REPORT SQRO-50-237/83152

The enclosed report provides details concerning the inoperability of
containment isolation valve 1-FCV-77-20 reactor coolant drain tank
nitrogen supply. This report is submitted in accordance with Sequoyah
unit 1 Technical Specification 6.9.1.13.b.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. J. Green

H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Sequoyah

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