NRC FORM 366 (7-77)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	I L Q A D 1 (2)	0 0 0 - 0 0 0 - 0 0 0 (3)	4 1 1 1 1 (4)	(5)
7	8	9 LICENSEE CODE 14	15 LICENSE NUMBER 25	26 LICENSE TYPE 30	57 CAT 58

CON'T

0	1	REPORT SOURCE L (6)	0 5 0 0 0 2 5 4 (7)	1 0 2 4 8 3 (8)	1 1 0 7 8 3 (9)
7	8	60	61 DOCKET NUMBER 68	69 EVENT DATE 74	75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 At 0735 hours, on October 24, 1983, while performing the Suppression Chamber to

0 3 Drywell Vacuum Breaker Monthly Exercise, QOS 1600-1, vacuum breaker 1-1601-32C

0 4 showed a dual indication on Division I while closing. As required by Technical

0 5 Specification 3.7.A.4.b, the Suppression Chamber to Drywell pressure separation was

0 6 performed immediately; and the vacuum breaker was verified to be closed. Since the

0 7 only apparent problem was a limit switch on Division I, the vacuum breaker was capable of performing its design function. The safety implications of this

0 8 occurrence were minimal.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	S A (11)	X (12)	X (13)	I N S T R U (14)	S (15)	Z (16)

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
8 3	—	0 43 43	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X (18)	X (19)	Z (20)	Z (21)	0 0 0 0	N (23)	N (24)	A (25)	M 3 0 2 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of this occurrence is suspected as being the Division I open limit switch

1 1 out of adjustment. The root cause will be determined at the next outage of

1 2 sufficient duration. A supplemental report will be issued following repairs.

1 3 Until that time, the Suppression Chamber to Drywell pressure separation test will be

1 4 performed every 15 days as required by Technical Specification 3.7.A.4.d.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	E (28)	0 8 9 (29)	NA	B (31)	Vacuum Breaker Inop Surveillance

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	NA

8312070110 831107
PDR ADOCK 05000254
S PDR

IE22 1/1

PUBLICITY

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)	NA	

NAME OF PREPARER S Reynolds PHONE 309-654-2241, ext 177

DMB

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NJK-83-410

November 7, 1983

J. Keppler, Regional Administrator
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-254, DPR-29, Unit One
Appendix A, Section 3.7.A.4.b

Enclosed please find Reportable Occurrence Report Number RO 83-43/03L-0 for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of Technical Specification 6.6.B.2.b, operation in a degraded mode permitted by a limiting condition for operation.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

NJK:DGC/bb

Enclosure

cc B. Rybak
A. Morrongiello
INPO Records Center