

CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)01 C A S O S 13 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CONT

01 REPORT SOURCE L 6 0 5 0 0 0 3 6 2 7 1 0 3 1 8 3 8 1 1 2 2 8 3 9  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On 10/31/83, at 1925, during a Unit 3 reactor trip from Mode 1, Aux-  
03 liary Feedwater Pump 3P-140 failed to start. The pump was manually re-  
04 set and started at 1936. Both electric Auxiliary Feedwater Pumps started  
05 and remained operable during this event. Public health and safety were  
06 not affected.  
07  
0809 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE  
H H 11 E 12 X 13 P U M P X X 14 B 15 Z 16  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100  
17 LER/RO REPORT NUMBER 8 3 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100  
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB PRIME COMP. SUPPLIER COMPONENT MANUFACTURER  
X 18 X 19 Z 20 Z 21 0 0 0 0 0 N 23 N 24 N 25 B 5 8 10 26  
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The pump was observed to be in the tripped condition. The cause of the  
11 trip is under investigation. In addition, intermittent failures of the  
12 trip indicator lights are being evaluated. Verification once per shift  
13 that the pump is not tripped is being performed pending completion of  
14 the evaluation.  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 10015 FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)  
X 28 0 6 2 29 NA A 31 Operator Observation  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 10016 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)  
Z 33 Z 34 NA NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 10017 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)  
0 0 0 37 Z 38 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 10018 PERSONNEL INJURIES NUMBER DESCRIPTION (41)  
0 0 0 40 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 10019 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)  
Z 42 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 10020 PUBLICITY ISSUED DESCRIPTION (45)  
N 44 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

NAME OF PREPARER

J. G. HAYNES

PHONE 714/492-7700

RECEIVED  
NRC

*Southern California Edison Company*

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

NOV 25 AM 11:11

SCE

REGIONAL VISE

J. G. HAYNES  
STATION MANAGER

November 22, 1983

TELEPHONE  
(714) 492-7700

U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region V  
1450 Maria Lane, Suite 210  
Walnut Creek, California 94596-5303

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-362  
14-Day Follow-Up Report  
Licensee Event Report No. 83-099  
San Onofre Nuclear Generating Station, Unit 3

Reference: Letter, J. G. Haynes (SCE) to J. B. Martin (NRC),  
dated November 9, 1983

The referenced letter provided you with confirmation of our prompt notification pursuant to Section 6.9.1.12.i of Appendix A, Technical Specifications to Facility Operating License NPF-15 for San Onofre Unit 3, of a reportable occurrence involving the Auxiliary Feedwater System.

On October 31, 1983, at 1925, Unit 3 was manually tripped from 62% reactor power in response to a loss of main feedwater. An Emergency Feedwater Actuation Signal (EFAS) was received when the unit was tripped, however, the Steam Driven Auxiliary Feedwater Pump 3P-140 failed to start. Pursuant to 10 CFR 50.72(a)7, at about 1955, the NRC Operations Center was notified of the reactor trip, EFAS actuation and the failure of 3P-140 to automatically initiate. Both electric driven Auxiliary Feedwater Pumps started and remained operable during this event.

Investigation into the failure of pump 3P-140, by the operator, revealed that the pump turbine was in a tripped condition. The mechanical linkage to the pump's turbine steam admission valve was manually reset, and while the EFAS signal was still present, the pump started at about 1936.

1/1

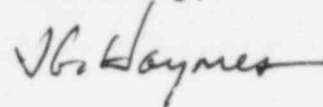
DE-22

November 22, 1983

The pump had previously been satisfactorily tested on October 30, 1983, at 1820. Although Control Room instrumentation is available to identify when the pump turbine is tripped, troubleshooting of the instrumentation subsequent to the plant trip on October 31, 1983, has indicated that there are intermittent failures of the instrumentation to indicate when the pump turbine is tripped. These intermittent failures are being investigated and appropriate corrective actions will be taken. The cause of the pump trip is unknown. No further spurious pump trips have occurred. An engineering program to evaluate the cause of the tripped condition of the pump is continuing and appropriate corrective actions will be taken. Pending completion of the program, visual verification that the 3P-140 overspeed trip mechanism is not in the tripped position will be performed once per shift.

If there are any questions regarding the above, please so advise.

Sincerely,



Enclosure: LER No. 83-099

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)  
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U. S. Nuclear Regulatory Commission  
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