



GPU Nuclear Corporation

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November 18, 1983

Dr. Thomas E. Murley, Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Dr. Murley:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Special Report 83-01T
Fire Suppression Water System Inoperability

This letter forwards three copies of the fourteen (14) day follow-up report to Special Report 83-01 in compliance with paragraph 3.12.B.3.b.3 of the Technical Specifications.

Very truly yours,

Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:RPJ:dam
Enclosures

cc: Director (40 copies)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Director (3)
Office of Management Information and
Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
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OYSTER CREEK NUCLEAR GENERATING STATION
Forked River, NJ 08731

Special Report 83-01T

Report Date

November 18, 1983

Occurrence Date

November 3, 1983

Identification of Occurrence

While fire pump 1-2 was out of service for maintenance, fire pump 1-1 was flow tested and did not meet the requirements of Technical Specifications, paragraph 4.12.B.1.e.1. These test results led to fire pump 1-1 being declared inoperable. With both fire pumps 1-1 and 1-2 inoperable, the fire suppression water system was considered inoperable.

While investigating this incident, it was discovered that the flow test operability requirements of Technical Specifications, paragraph 4.12.B.1.e.1, had not been met for fire pump 1-1 since the incorporation of these requirements into the Technical Specifications in December, 1981.

These events require a Special Report to the Commission as defined in the Technical Specifications, paragraph 3.12.B.3.b.

Conditions Prior to Occurrence

Plant is in an outage condition with no fuel in the vessel.

Description of Occurrence

On November 3, 1983, the Fire Suppression Water System at Oyster Creek was declared inoperable based on the following conditions:

1. Fire pump 1-2 had been taken out of service on October 22, 1983 at approximately 1045 hours. Smoke was observed emitting from under the pumphead during a post-maintenance in-service test.
2. Fire pump 1-1 was determined inoperable on November 3, 1983. The pump test recorded the pump discharge pressure at 135 psig for 2006 GPM. Technical Specifications require 2000 GPM at a system head of 360 feet.

Apparent Cause of Occurrence

The gearbox antirotation device of fire pump 1-2 required machining. This was the apparent cause of inoperability for pump 1-2.

The inoperability of fire pump 1-1 can be attributed to not identifying pump inoperability on past functional tests. The acceptance criteria in surveillance procedure 645.6.012, "Fire Pump Functional Test" was not properly revised to reflect the Technical Specification surveillance requirements which were issued via Amendment 58 on December 21, 1981. The procedure required comparison of flow and pressure test data against the design rated curve of the pump. Significant deviation from the design curve was used as the criterion for inoperability, rather than the Technical Specification requirement of 2000 GPM at a system head of 360 feet.

It is believed that fire pump 1-1 relief valve (set at 150 PSIG) does not allow the pump to achieve the required pressure. Although pump 1-2 also has a relief valve setting of 150 PSIG, it has met the Technical Specification requirements for operability on previous functional tests. This may be because of relief point variation.

Corrective Action

Fire pump 1-2 was declared operable at 1830 hours on November 9, 1983. The antirotation device was machined to specified tolerances correcting the overheating and smoking condition observed on October 22, 1983.

A modification is underway to install new pressure relief valves on both fire pumps 1-1 and 1-2. It is expected that the new relief valve on fire pump 1-1 will allow the system to obtain a discharge pressure that will be in compliance with the Technical Specifications. Should this modification not rectify the situation, further corrective action will be taken within thirty days.

The acceptance criteria in procedure 645.6.012 has been revised to incorporate the requirements in Section 4.12.B of the Oyster Creek Technical Specifications.

In addition, a complete review of all Surveillance procedures has been conducted to ensure that the procedures reflect Technical Specification requirements.