

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | 1 | L | Q | A | D | 1 | 2 | 0 | 0 | 0 | - | 0 | 0 | 0 | - | 0 | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

CON'T

0 1 | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 5 | 4 | 7 | 1 | 0 | 2 | 6 | 8 | 3 | 8 | 1 | 1 | 1 | 5 | 8 | 3 | 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32
REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | On October 26, 1983, while performing the Pressure Suppression Chamber and
0 3 | Reactor Building Vacuum Breaker Actuation Calibration Test, QIS 48-1, Pressure
0 4 | Switch number two of PSI-1-1622A actuated at 0.52 psid. This is 0.02 psid
0 5 | greater than the Technical Specification 3.7.A.3.a limit. The consequences
0 6 | were minimal because the 100 percent redundant vacuum breaker system was
0 7 | operable. The Suppression Chamber is designed for 2 psig external pressure;
0 8 | therefore, both vacuum breakers would have actuated prior to reaching this limit.
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

0 9 | 1 | B | 11 | X | 12 | Z | 13 | I | N | S | T | R | U | 14 | S | 15 | Z | 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE
17 LER-RO REPORT NUMBER 18 EVENT YEAR 19 SEQUENTIAL REPORT NO. 20 OCCURRENCE CODE 21 REPORT TYPE 22 REVISION NO.
23 ACTION TAKEN 24 FUTURE ACTION 25 EFFECT ON PLANT 26 SHUTDOWN METHOD 27 HOURS 28 ATTACHMENT SUBMITTED 29 NPRD-4 FORM SUB 30 PRIME COMP. SUPPLIER 31 COMPONENT MANUFACTURER
32 E | 18 | Z | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | 22 | N | 23 | N | 24 | N | 25 | S | 3 | 8 | 2 | 26
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The cause of this occurrence was instrument setpoint drift. Pressure Switch
1 1 | PS-1-1622A was immediately recalibrated to within the Technical Specification
1 2 | limits. There has been past occurrences of setpoint drift with these pressure
1 3 | switches, the most recent being documented under LER 81-14/03L-0, on August
1 4 | 17, 1981.
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

1 5 | E | 28 | 0 | 9 | 0 | 29 | NA | 30 | B | 31 | Routine Testing | 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33
FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION
1 6 | Z | 33 | Z | 34 | NA | 35 | NA | 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE
1 7 | 0 | 0 | 0 | 37 | Z | 38 | NA | 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION
1 8 | 0 | 0 | 0 | 40 | NA | 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33
PERSONNEL INJURIES NUMBER DESCRIPTION
1 9 | Z | 42 | NA | 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION
2 0 | N | 44 | NA | 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33
PUBLICITY ISSUED DESCRIPTION
2 0 | N | 44 | NA | 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33

NAME OF PREPARER P Todd

PHONE: 309-654-2241, ext 193

8312020325 831115
PDR ADDCK 05000234
S PDR

NRC USE ONLY



Commonwealth Edison

Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

DMB

NJK-83-423

November 15, 1983

J. Keppler, Regional Administrator
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-254, DPR-29, Unit One
Appendix A, Section 3.7.A.3.a

Enclosed please find Reportable Occurrence Report Number RO 83-44/03L-0
for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of
Technical Specification 6.6.B.2.a, engineered safety feature instrument
setting found less conservative than established by Technical Specifica-
tions but which does not prevent the fulfillment of the functional
requirements of affected systems.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

N. J. Kalivianakis
Station Superintendent

NJK:DGC/bb

Enclosure

cc B. Rybak
A. Morrongiello
INPO Records Center

NOV 21 1983

IE22-11