MONTHLY OPERATIONS REPORT

SEPTEMBER 1983

Throughout the report period, the Oyster Creek Station remained shutdown for the 1983 Refueling/Maintenance Outage.

Processing of new reactor fuel commenced during the report period.

On September 20, 1983, Reactor Building Ventilation System isolation valve V-28-12 failed to fully close during routine testing. V-28-12 was repaired and returned to an operable status on September 28. The problem was attributed to leaking piston seal rings in the valve actuator. Consequently, the Operations Department has requested that all Reactor Building Ventilation System valves be inspected for this problem and repaired as necessary. These repairs should reduce the demand on the plant's Control Air System and thus reduce the number of times the air compressors cycle to maintain system pressure.

The only problem experienced with plant air compressors during the report period was with No. 2 air compressor. It was out of service for a two day period due to a starting and loading problem.

Repairs were started on several leaks in Condensate Transfer System piping. These repairs are expected to be completed and the system made available by October 1, 1983.

On September 7 and 8, outage work in the drywell and torus was temporarily stopped due to personnel experiencing heat distress as a result of high ambient air temperature. Plant Engineering is addressing this problem with assistance from the Plant Operations and Maintenance and Construction Departments.

A scram signal (half scram) was received in Reactor Protection System No. 2 due to the failure of one of the associated high drywell pressure sensor (RE 04B) relays (2k9). The relay was replaced.

A half scram was received due to a detector failure on "D" main steam line radiation sensor. A new detector will be installed.

The following items were also considered noteworthy:

1. Dilution pump No. 1 tripped due to a failed seal water pressure switch.

2. Faulty contacts in the Rotary Inverter for Continuous Instrument Panel No. 3 caused a voltage regulator problem which made its associated auto-bus-transfer switch (IT-3) transfer periodically between the normal and alternate power supply. The Rotary Inverter was placed back in service after the contacts were replaced. It is presently being monitored for satisfactory operation.

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3. Fire diesel pump No. 2 was out of service for a two day period due to an oil leak.

4. The bearings on "D" Augmented Fuel Pool Cooling pump were replaced.

5. The Stack Gas Sample System isokinetic probe was removed for decon, inspection, and modification. A temporary probe has been installed.

6. Reactor Building Closed Cooling Water (RBCCW) pump No. 1 was out of service for a one day period due to a breaker tripping problem.

7. On September 21, the fuel pool water spilled into the ventilation system while backwashing the fuel pool filter. The cause of the event was due to a combination of procedure deficiencies, equipment malfunction and operator error. The procedure will be revised, and the critique discussed with operators.

8. As a result of work activities in the 460 Volt Switchgear Room and Cable Spreading Room, two fire barriers, which had been breached, were repaired. Fire watches were posted in the interim.

9. Inspection of main steam reheaters (RHTR) 1-2 and 1-3 was completed. One tube leak in RHTR 1-2 was identified.

The following Licensee Event Report was submitted during September 1983:

Reportable Occurrence No. 50-219/83-17/01T:

On August 16, 1983, Diesel Generator (DG) No. 1 tripped during a fast start test. DG No. 1 was then declared inoperable. After a thorough review of the test data, it was determined on August 26, 1983 that DG No. 2 might have the potential for the same problem. At that time, DG No. 2 was also taken out of service. The cause of the occurrence was attributed to close timing between the Main Circuit Breaker Auxiliary Relay and the DG's undervoltage sensing protective relay. Following a change in the time delay setpoint (from 4 seconds to 7 seconds) both DGs were returned to service on August 27, 1983.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH September 1983

DOCKET NO. 50-219
UNIT NAME Oyster Creek
DATE Oct. 1, 1983
COMPLETED BY R. Baran
TELEPHONE 4640

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	License Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
31	2-11-83	S	5544	C	1	NA	ZZ	ZZZZZZ	Start of the 1983 Refuel/ Maintenance Outage.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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OPERATING DATA REPORT
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: September, 1983
3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe): 687.5 X 0.8 = 550
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe):
11. REASON FOR RESTRICTION, IF ANY: NONE

		MONTH	YEAR	CUMULATIVE
12.	REPORT PERIOD HRS	720.0	6,551.0	120,719.0
13.	HOURS RX CRITICAL	0.0	1,009.6	84,622.8
14.	RX RESERVE SHTDWN HRS	0.0	0.0	468.2
15.	HRS GENERATOR ON-LINE	0.0	1,007.8	82,693.6
16.	UT RESERVE SHTDWN HRS	0.0	0.0	0.0
17.	GROSS THERM ENER (MWH)	0.0	853,300.0	136,224,730.5
18.	GROSS ELEC ENER (MWH)	0.0	244,630.0	46,056,905.0
19.	NET ELEC ENER (MWH)	-1,461.0	210,583.0	44,291,111.0
20.	UT SERVICE FACTOR	0.0	15.4	68.5
21.	UT AVAIL FACTOR	0.0	15.4	68.5
22.	UT CAP FACTOR (MCD NET)	0.0	5.2	59.2
23.	UT CAP FACTOR (DER NET)	0.0	4.9	56.4
24.	UT FORCED OUTAGE RATE	0.0	0.0	9.7
25.	FORCED OUTAGE HRS	0.0	0.0	8,916.8

26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION): N/A
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME: 02/10/84

AVERAGE DAILY POWER LEVEL
NET MWe

DOCKET #. 50-219
UNIT. . . . . .Oyster Creek #1
REPORT DATE . . . . . .OCTOBER 03, 1983
COMPILED BY . . . . . .JOSEPH R. MOLNAR
TELEPHONE # 609-971-4699

MONTH SEPTEMBER, 1983

DAY	MW	DAY	MW
1.	0	16.	0
2.	0	17.	0
3.	0	18.	0
4.	0	19.	0
5.	0	20.	0
6.	0	21.	0
7.	0	22.	0
8.	0	23.	0
9.	0	24.	0
10.	0	25.	0
11.	0	26.	0
12.	0	27.	0
13.	0	28.	0
14.	0	29.	0
15.	0	30.	0

Oyster Creek Station #1
Docket No. 50-219

REFUELING INFORMATION - September, 1983

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: Presently shutdown for Refueling

Scheduled date for restart following refueling: February 10, 1984

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Technical Specification Change Request No. 96 was submitted on August 31, 1982 for incorporation of GE assemblies into the Cycle 10 core.

Scheduled date(s) for submitting proposed licensing action and supporting information:

October 30, 1983 - The final supplement to the reload analysis, delineating the specific core configuration for Cycle 10 operation, will be submitted.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a date.

2. Exxon Fuel Assemblies - no major changes have been nor are there any anticipated.

The number of fuel assemblies (a) in the core = 0
(b) in the spent fuel storage pool = 1361

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800 Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capapcity:

Full core offload capability will be lost after the 1985 outage. Batch discharge capability will be lost after the 1987 outage. Expanded spent fuel pool rack capacity (2,600) is scheduled for 1984.

GPU Nuclear

GPU Nuclear Corporation
Post Office Box 388
Route 9 South
Forked River, New Jersey 08731-0388
609 971-4000
Writer's Direct Dial Number:

October 14, 1983

Director
Office of Management Information
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Mr. Michael Laggart at (609) 971-4643.

Very truly yours,

Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:PFC:dam
Enclosures

cc: Director (10)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Regional Administrator (1)
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731

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