

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 P A T M I 1 200-000000-000 411111 5
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 37 CAT 58

CON'T
01 L 05000289 709288 38 102883 9
7 8 9 REPORT SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

02 While in hot shutdown conditions for non-critical steam generator testing, the
03 reactor building high pressure switch setpoint on RPS channel "A" was found during
04 a surveillance to be 4.2 psig which is less conservative than the 4.0 psig limit
05 specified in T.S. Table 2.3-1. This is considered reportable under T.S.
06 6.9.2.B.(1). Public health and safety were unaffected. The remaining three
07 channels were within Technical Specification limits.
08
09

09 I A E E I N S T R U S Z
7 8 9 SYSTEM CODE 9 10 CAUSE CODE 11 CAUSE SUBCODE 12 COMPONENT CODE 13 COMP. SUBCODE 14 VALVE SUBCODE 15

17 83 033 03 L 0
7 8 9 LER/RO REPORT NUMBER 21 22 EVENT YEAR 23 24 SEQUENTIAL REPORT NO. 25 26 OCCURRENCE CODE 27 28 REPORT TYPE 29 30 REVISION NO. 31 32

E Z Z Z 0000 Y N N 5382
33 34 ACTION TAKEN 35 FUTURE ACTION 36 EFFECT ON PLANT 37 SHUTDOWN METHOD 38 HOURS 39 ATTACHMENT SUBMITTED 40 NPD-4 FORM SUB. 41 PRIME COMP. SUPPLIER 42 COMPONENT MANUFACTURER 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

10 Cause was setpoint drift. The pressure switch was recalibrated to trip at a
11 setpoint within Technical Specifications. An entry was made in the machinery
12 history. No further action is planned at this time but additional corrective
13 action may be indicated through future routine surveillance.
14

15 G 0000 N/A B Inservice Inspection
7 8 9 FACILITY STATUS 10 % POWER 11 OTHER STATUS 12 METHOD OF DISCOVERY 13 DISCOVERY DESCRIPTION 14

16 Z Z N/A N/A
7 8 9 ACTIVITY CONTENT 10 RELEASED OF RELEASE 11 AMOUNT OF ACTIVITY 12 LOCATION OF RELEASE 13

17 000 N/A
7 8 9 PERSONNEL EXPOSURES 10 NUMBER 11 DESCRIPTION 12

18 000 N/A
7 8 9 PERSONNEL INJURIES 10 NUMBER 11 DESCRIPTION 12

19 Z 8311170235 831028 N/A
7 8 9 LOSS OF OR DAMAGE TO FACILITY 10 TYPE 11 DESCRIPTION 12

20 N PDR ADOCK 05000289 N/A
7 8 9 PUBLICITY 10 ISSUED 11 DESCRIPTION 12

20 N PDR N/A
7 8 9 NRC USE ONLY 10

NAME OF PREPARER M. R. Knight PHONE 717-948-8647

I. CURRENT ACTIVITIES AT THE TIME OF THE OCCURRENCE

TMI Unit 1 was in a Hot Shutdown condition, performing Reactor Protection System high building pressure channel surveillance per SP 1302-5.7.

II. CIRCUMSTANCES LEADING TO THE OCCURRENCE

On September 28, 1983, while performing SP 1302-5.7 on RPS channel "A", the trip setpoint for the reactor building pressure switch was found to be less conservative than the limit specified in Technical Specification Table 2.3-1. Actual trip setpoint was 4.2 psig; Table 2.3-1 limit is 4.0 psig.

III. DESCRIPTION OF OCCURRENCE

The setpoints for the remaining three channels were within Technical Specification limits. This item is considered reportable under Tech. Spec. 6.9.2.B.(1) as an instrument setpoint which is less conservative than that required by the Tech. Spec., but which does not prevent the fulfillment of the functional requirements of the affected system.

IV. RESULTANT EVENTS

There were no resultant events. The plant was in a hot shutdown condition, for which operability of this equipment is not required. The remaining three channels were tested and found to trip within Technical Specification limits.

V. PREVIOUS EVENTS OF A SIMILAR NATURE

None.

VI. ROOT CAUSE OF OCCURRENCE

Pressure switch setpoint drifted.

VII. IMMEDIATE CORRECTIVE ACTION

The pressure switch was recalibrated to trip satisfactorily at a setpoint within the Technical Specification limit and a notation was made in machinery history.

VIII. LONG TERM CORRECTIVE ACTION

No further action is planned at this time, however, additional corrective action may be indicated through future routine surveillance.

IX. COMPONENT FAILURE DATA

Static O Ring Model 12NK45



GPU Nuclear Corporation
Post Office Box 480
Route 441 South
Middletown, Pennsylvania 17057-0191
717 944-7621
TELEX 84-2386
Writer's Direct Dial Number:

October 28, 1983
5211-83-317

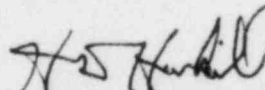
Dr. Thomas E. Murley
Region I, Regional Administrator
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Sir:

Three Mile Island Nuclear Station, Unit 1 (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
LER 83-033/03L-0

This letter transmits Licensee Event Report (LER) No. 83-033/03L-0 which deals with setpoint drift of a pressure switch. Public health and safety were unaffected.

Sincerely,


H. D. Hukill
Director, TMI-1

HDH:MRK:vjf

Attachments

cc: R. Conte
Document Management Branch