

LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

| | | | | | | | | | | | | | | | | | | | | | | | | | | | | | | |
|---|---|---------------|---|---|---|---|---|----|----|----------------|---|---|---|---|---|---|---|---|---|----|----|--------------|---|---|---|---|----|----|--------|--|
| 0 | 1 | C | T | M | N | S | 2 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | | | 5 | |
| 8 | 9 | LICENSER CODE | | | | | | 14 | 15 | LICENSE NUMBER | | | | | | | | | | 25 | 26 | LICENSE TYPE | | | | | 30 | 57 | CAT 58 | |

REPORT SOURCE L 6 0 5 0 0 0 3 3 6 7 0 6 3 0 8 3 8 1 0 2 8 8 3 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | Non destructive examinations of the core support barrel (CSB) reveal that a
thru-wall crack 10.2 inches in length is present at the number 4, lug area.
0 3 |
Further testing is in progress on all remaining lug areas and a final
0 4 |
inspection report with expected corrective measures to be taken will be
0 5 |
provided in a further update to this report.

| | | | | | | | | | | | | | | | |
|------------------------|--|-------------------------|--|-----------------------------|--|---------------------------|--|------------------------------------|--|--------------------------------|--|---------------------------|--|--------------------------------|--|
| 09 | | SYSTEM CODE RA (11) | | CAUSE CODE E (12) | | CAUSE SUBCODE B (13) | | COMPONENT CODE V E S S E L (14) | | | | COMP SUBCODE A (15) | | VALVE SUBCODE Z (16) | |
| 17 | | EVENT YEAR 83 | | SEQUENTIAL REPORT NO 024 | | OCCURRENCE CODE 01 | | REPORT TYPE X | | REVISION NO. 1 | | | | | |
| ACTION TAKEN X (18) | | FUTURE ACTION X (19) | | EFFECT ON PLANT C (20) | | SHUTDOWN METHOD Z (21) | | HOURS 0000 (22) | | ATTACHMENT SUBMITTED N (23) | | NPRD-4 FORM SUB Y (24) | | PRIME COMP. SUPPLIER N (25) | |
| 33 | | 34 | | 35 | | 36 | | 37 | | 41 | | 42 | | 43 | |
| 44 | | 45 | | 46 | | 47 | | 48 | | 49 | | 50 | | 51 | |

| CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) | |
|---|---|
| 1 0 | The cause of the damage to the CSB is unknown. The repairs to the CSB will be de- |
| 1 1 | termined when the full inspection has been completed and those details also |
| 1 2 | provided in the update report. |

6 7 8 9
FACILITY STATUS (1) 5 (H) (28) % POWER (0) (0) (0) (29) OTHER STATUS (30) NA METHOD OF DISCOVERY (31) Inservice Inspection DISCOVERY DESCRIPTION (32)

7 8 9 10 11 12 13 14 15 16 17 18 19 20
ACTIVITY CONTENT RELEASED OF RELEASE (1) 6 (Z) (33) (Z) (34) AMOUNT OF ACTIVITY (35) NA LOCATION OF RELEASE (36)

7 8 9 10 11 12 13 14 15 16 17 18 19 20
PERSONNEL EXPOSURES NUMBER (1) 7 (0) (0) (0) (37) (Z) (38) DESCRIPTION (39) NA 8311170146 831028 PDR ADOCK 05000336 S PDR

7 8 9 10 11 12 13 14 15 16 17 18 19 20
PERSONNEL INJURIES NUMBER (1) 8 (0) (0) (0) (40) DESCRIPTION (41) NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20
LOSS OF OR DAMAGE TO FACILITY TYPE (1) 9 (Z) (42) DESCRIPTION (43) NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20
PUBLICITY ISSUED (2) 0 (N) (44) DESCRIPTION (45) NA NRC USE ONLY

NAME OF PREPARER R. W. Bates

PHONE 203-447-1791 X 4470

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

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October 28, 1983
MP-5478

Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Reference: Facility Operating License DPR-65
Docket No 50-336
Reportable Occurrence RO-50-336/83-24/01-X-1

Dear Dr. Murley:

This letter forwards updated Licensee Event Report 83-24/01-X-1 required to be submitted pursuant to the requirements of Millstone Unit 2 Appendix A Technical Specifications, Section 6.9.1.8.i, discovery of conditions not specifically considered in the Safety Analysis Report or Technical Specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition. An additional three copies of the report are enclosed.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

A handwritten signature in cursive script, appearing to read 'E. J. Mroczka'.

E. J. Mroczka
Station Superintendent
Millstone Nuclear Power Station

EJM/RWB:ejl

Attachment: LER RO-50-366/83-24/01-X-1

cc: Director, Office of Inspection and Enforcement Washington, D. C. (30)

Director, Office of Management Information and Program Control,
Washington, D.C. (3)

U.S. Nuclear Regulatory Commission, c/o Document Management Branch,
Washington, D.C. 20555

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