

LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N Y N M P 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T

0 1 REPORT SOURCE L 6 0 5 0 0 0 2 2 0 7 0 9 0 9 8 3 8 0 9 2 8 8 3 9
7 8 60 61 DOCKET NUMBER 65 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During normal operations while performing surveillance test N1-ISP-RVT on the main
0 3 steam safety valve tailpipe temperature sensors, the inputs indicated excessive drift
0 4 across thermocouple B410. This places the plant in a limiting condition for operation
0 5 of Technical Specification 3.6.11. This is of minimal safety concern since the
0 6 thermocouples act as backup sensors to the primary acoustic sensors which are currently
0 7 operable. Similar problems were reported in LERs 83-15 and 81-52.

0 8 _____ 80

0 9 SYSTEM CODE C C 11 CAUSE CODE X 12 CAUSE SUBCODE Z 13 COMPONENT CODE I N S T R U 14 COMP. SUBCODE E 15 VALVE SUBCODE Z 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20
17 LER/RO REPORT NUMBER 8 3 21 22 EVENT YEAR 23 24 SEQUENTIAL REPORT NO. 0 2 6 25 26 27 OCCURRENCE CODE 0 3 28 29 REPORT TYPE L 30 31 REVISION NO. 0 32
ACTION TAKEN Z 18 FUTURE ACTION X 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 22 ATTACHMENT SUBMITTED N 23 NPRD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER N 25 COMPONENT MANUFACTURER P 0 5 0 26
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of the thermocouples drifting is not known, however, it is suspected that
1 1 it could be attributed to a poor connection in the terminal box. The corrective
1 2 action necessary to restore the thermocouples to normal operation will be determined
1 3 during the next cold shutdown when there is access to the drywell.

1 4 _____ 80

1 5 FACILITY STATUS E 28 % POWER 0 9 9 29 OTHER STATUS NA 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Test results 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 6 ACTIVITY CONTENT Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE 0 38 DESCRIPTION NA 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

2 0 PUBLICITY ISSUED N 44 DESCRIPTION 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

8311070190 830930
PDR ADDOCK 05000220
S PDR

NRC USE ONLY

NAME OF PREPARER D.A. DeLorenzo

PHONE: (315) 349-2616

September 30, 1983

Dr. Thomas E. Murley
Regional Administrator
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

RE: Docket No. 50-220
LER 83-26

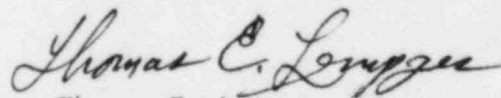
Dear Dr. Murley,

In accordance with Nine Mile Point Nuclear Station Unit #1 Technical Specifications, we hereby submit the following Licensee Event Report:

83-26 which is being submitted in accordance with Section 6.9.2.(b)(2), conditions leading to operation or plant shutdown required by a limiting condition for operation.

This report was completed in the format designated in NUREG-0161, dated July 1977.

Very truly yours,



Thomas E. Lempges
Vice President
Nuclear Generation

TEL/jm

Attachments (3 copies)

cc: Director, Office of I&E (30 copies)
Director, Office of MIPC (3 copies)

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