

SUPPLEMENTARY INFORMATION

REPORT NO.: 50-302/83-034/01T-1

FACILITY: Crystal River Unit 3

REPORT DATE: October 10, 1983

OCCURRENCE DATE: August 12, 1983

IDENTIFICATION OF OCCURRENCE:

Both Reactor Coolant Pump Power Monitor (RCPPM) channels associated with reactor coolant pump RCP-1C were bypassed. Action Statement 25 of Technical Specification 3.3.1.1, Table 3.3.1 allows one less than the "minimum channels required" to be operable. However, T.S. 3.3.1.1 requires both channels to be operable. Thus, this condition exceeded the appropriate Action Statement.

CONDITIONS PRIOR TO OCCURRENCE:

Mode 1 (74% power); 3 pump operation.

DESCRIPTION OF OCCURRENCE:

While testing the RCP-1C's with RCP-1C inoperable and in the tripped condition, both RCP-1C channels from RCP-1C were bypassed to avoid tripping the plant. This condition existed for approximately 1.3 hours at which time these channels were returned to their normal (un-bypassed) condition.

DESIGNATION OF APPARENT CAUSE:

The use of an inadequate procedure caused this occurrence in that some steps within Surveillance Procedure SP-110 would not satisfy the least conservative Limiting Condition for Operation of T.S. 3.3.1.1. (The source of the original inadequacy is unknown.) While revising SP-110 to include three pump operation, this condition was not recognized and, thus, approval was given to revise the procedure as proposed.

ANALYSIS OF OCCURRENCE:

Babcock & Wilcox has completed analysis to support operation of CR-3 without the RCP-1C circuitry for the current cycle. This analysis relies upon revised RC flow assumptions, revised error analyses, deletion of rod-bow penalty, utilization of cycle specific peaking factors, and some revision of operating limits. Furthermore, the only coastdown events not protected by the RCP-1C circuitry with one pump not operating and bypassed are the single pump trip events. Loss of more than one pump would still have caused an RCP-1C trip.

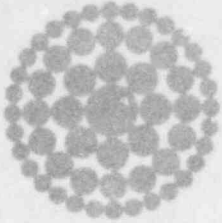
Therefore, the combination of an available analysis (to be docketed as Change Request 82, Attachment "B", later this year) and the relatively low probability of a specific coastdown during a short period of time is considered adequate to show that it is unlikely that this event would have caused any adverse impact on public health and safety.

CORRECTIVE ACTION:

Authorization to bypass both channels has been received from Region II with NRR concurrence. The internal procedure revision process has been modified to require that a reviewer holding a SRO license verifies that the affected Technical Specification is satisfied. Further checks are under investigation.

FAILURE DATA:

This is the first occurrence of this type and the sixteenth report under Technical Specification 3.3.1.1.



**Florida
Power**
CORPORATION

USNRC REGION II
ATLANTA, GEORGIA

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October 10, 1983
3F-1083-10

Mr. James P. O'Reilly
Regional Administrator, Region II
Office of Inspection & Enforcement
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, GA 30303

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
Licensee Event Report No. 83-034, Rev. 1

Dear Mr. O'Reilly:

Enclosed is Licensee Event Report No. 83-034, Revision 1 and the attached supplementary information sheet, which are submitted in accordance with Technical Specification 6.9.1.8(b). This report updates our previous submittal of September 9, 1983.

Should there be any questions, please contact this office.

Sincerely,

G.R. Westafer
Manager
Nuclear Operations Licensing & Fuel Management

AEF:feb

Enclosure

cc: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

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Docket No. 50-263

Northern States Power Company
ATTN: Mr. C. E. Larson
Director of Nuclear
Generation
414 Nicollet Mall
Minneapolis, MN 55401

Gentlemen:

This refers to the routine safety inspection conducted by Mr. C. Brown of this office on August 2 through September 1, 1983, of activities at the Monticello Nuclear Generating Station authorized by NRC Operating License No. DPR-22 and to the discussion of our findings with Mr. W. Shamla at the conclusion of the inspection.

The enclosed copy of our inspection report identifies areas examined during the inspection. Within these areas, the inspection consisted of a selective examination of procedures and representative records, observations, and interviews with personnel.

No items of noncompliance with NRC requirements were identified during the course of this inspection.

In accordance with 10 CFR 2.790(a), a copy of this letter and the enclosure(s) will be placed in the NRC Public Document Room unless you notify this office, by telephone, within ten days of the date of this letter and submit written application to withhold information contained therein within thirty days of the date of this letter. Such application must be consistent with the requirements of 2.790(b)(1). If we do not hear from you in this regard within the specified periods noted above, a copy of this letter and the enclosed inspection report will be placed in the Public Document Room.

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