U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Field	Value
01 LICENSEE CODE (2)	ILQAD1
LICENSE NUMBER (3)	000-000-000
LICENSE TYPE (4)	4 1 1 1 1
CAT (5)	
CON'T 01 REPORT SOURCE (6)	L
DOCKET NUMBER (7)	05000254
EVENT DATE (8)	090883
REPORT DATE (9)	100383

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

On September 8, 1983, the LPCI loop select for the 'A' RHR loop was declared inoperable due to the inability to close the 1A Recirculation Pump Discharge valve MO 1-202-5A. The consequences of this occurrence are minimized because all subsystems were demonstrated operable as required by Technical Specification 3.5.A.5.

SYSTEM CODE (11)	CAUSE CODE (12)	CAUSE SUBCODE (13)	COMPONENT CODE (14)	COMP SUBCODE (15)	VALVE SUBCODE (16)
SF	E	A	ELECON	Z	Z

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
83	—	034	/	03	L	—	0

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER (26)
A	Z	B	Z	0143	Y	Y	N	G080

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

The control cables for MO 1-202-5A shorted, causing the control transformer for the breaker to short. The valve was closed by using a spare cubicle from Unit Two. The cables were replaced during a Maintenance Outage on September 17, 1983, and MO 1-202-5A was tested satisfactorily on September 18, 1983.

FACILITY STATUS (28)	% POWER (29)	OTHER STATUS (30)	METHOD OF DISCOVERY (31)	DISCOVERY DESCRIPTION (32)
E	098	NA	A	Operational Event

ACTIVITY RELEASED (33)	CONTENT OF RELEASE (34)	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
Z	Z	NA	NA

PERSONNEL EXPOSURES

NUMBER (37)	TYPE (38)	DESCRIPTION (39)
000	Z	NA

PERSONNEL INJURIES

NUMBER (40)	DESCRIPTION (41)
000	NA

LOSS OF OR DAMAGE TO FACILITY (43)

TYPE (42)	DESCRIPTION
Z	

8310170491 831003
PDR ADOCK 05000254
S PDR

IE22 1/1

PUBLICITY

ISSUED (44)	DESCRIPTION (45)	NRC USE ONLY
N	NA	

NAME OF PREPARER C Iben PHONE 309-654-2241, ext 173

I. LER NUMBER: LER 83-34/03L

II. LICENSEE NAME: Commonwealth Edison Company
Quad-Cities Nuclear Power Station

III. FACILITY NAME: Unit One

IV. DOCKET NUMBER: 050-254

V. EVENT DESCRIPTION:

On September 8, 1983, at 6:50 a.m., Unit One was in the RUN mode operating at 797 MWe and 2482 MWt. At 6:50 a.m., the Control Room was notified that smoke was emanating from the motor control center cubicle that houses the 1A Recirculation pump discharge valve, MO 1-202-5A. An Equipment Operator was dispatched to MCC 18/19-5 and discovered the breaker for the MO 1-202-5A valve was tripped, and that the control transformer had been the source of the smoke due to overheating.

The overheating of the control transformer had damaged the transformer to such an extent that the Equipment Operator could not operate the breaker locally. Valve MO 1-202-5A was then taken out of service in the OPEN position. This action rendered the LPCI loop select of RHR for the 'A' Recirculation loop inoperable. The required surveillance tests were immediately initiated in accordance with Technical Specification 4.5.A.5.

On September 9, 1983, the MO 1-202-5A valve was closed by utilizing a similar breaker from Unit Two. The valve was taken out of service and thus, LPCI mode of RHR was declared operable. Single loop operation was in affect until September 15, 1983, when the unit was shutdown for a scheduled Maintenance Outage. The required surveillances for single loop operation were performed in accordance with the Facility Operating License, 3.k.

Work Request Q28250 was initiated to investigate the cause of the occurrence.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

Immediately after the LPCI mode of RHR was declared inoperable, the Containment Cooling mode of RHR, both standby Diesel Generators and both Core Spray subsystems were demonstrated to be operable.

Since all required system operability surveillances were performed satisfactorily, safe operation of the plant was maintained at all times.

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VII. CAUSE:

The LPCI mode of RHR had become inoperable due to the inability to operate the MO 1-202-5A valve. Further investigation revealed that the control cables for the valve had faulted at some point between the X-105B Drywell Penetration and the valve. Work Request Q28302 was initiated to remove the faulted cables.

A definitive root cause of the faulted cable is not possible to ascertain. It can only be surmised that the cable insulation became slightly damaged during initial installation, and over the years became exposed to the point that a fault occurred.

A similar incident with this valve occurred on March 13, 1981, and is documented in LER/RO 81-07/03L. Corrective action at that time included the installation of a temporary cable. During the fall 1982 Refuel Outage, spare cable in the same conduit was connected to the valve motor. These spare cables became available when a motor-operated bypass valve was removed from the Recirculation System.

VIII. CORRECTIVE ACTION:

The initial corrective action taken entailed the closing of the 'A' Recirculation discharge valve. The LPCI mode of RHR was declared inoperable at that time. During the scheduled Maintenance Outage, the faulted cables were removed from the conduit and new cables were installed. Five cables were removed which included two from the original -5A valve cables and three from the old -7A bypass valve cabling. All other cables in the same conduit were meggered and tested satisfactorily. Valve operability tests were performed to verify the proper logic circuits were intact on September 18, 1983.

DMB

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NJK-83-347

October 3, 1983

J. Keppler, Regional Administrator
Office of Inspection and Enforcement
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reference: Quad-Cities Nuclear Power Station
Docket Number 50-254, DPR-29, Unit One
Appendix A, Sections 3.5.A.3 and 6.6.B.2.b

Enclosed, please find Reportable Occurrence Report Number RO 83-34/03L for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of Technical Specification 6.6.B.2.b, as a condition leading to operation in a degraded mode permitted by a limiting condition for operation.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

L. J. Gerner for
N. J. Kalivianakis
Station Superintendent

NJK:JRW/bb

Enclosure

cc B. Rybak
A. Morrongiello
INPO Records Center

OCT 11 1983

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