

U. S. NUCLEAR REGULATORY COMMISSION

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

SYSTEM CODE R H 11		CAUSE CODE E 12		CAUSE SUBCODE B 13		COMPONENT CODE P U M P X X 14				COMP. SUBCODE B 15		VALVE SUBCODE Z 16	
7 8		9 10		11 12		13 14 15 16 17 18				19 20		21 22	
LER/RO REPORT NUMBER 17		EVENT YEAR 8 3 21 22		SEQUENTIAL REPORT NO. 0 1 1 23 24 25 26		OCCURRENCE CODE / 27		0 3 28 29		REPORT TYPE X 30		REVISION NO. 1 31 32	
ACTION TAKEN A 18		FUTURE ACTION Z 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 0 22 23 24 25		ATTACHMENT SUBMITTED Y 23		NPRD-4 FORM SUB. Y 24	
33 34		35 36		37 38		39 40		41 42		43 44		45 46 47 48	
PRIME COMP. SUPPLIER N 25		COMPONENT MANUFACTURER P 0 2 5 26											

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

8 9

FACILITY STATUS		% POWER			OTHER STATUS (30)		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION (32)		
1	5	E	23	1	0	0	29	N/A	B	31	Routine inservice testing

10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)	Z	(38)	N/A	(39)

		LOSS OF OR DAMAGE TO FACILITY		
		TYPE	DESCRIPTION	(43)
1	9	Z	N/A	(42)

PUBLICITY
 ISSUED DESCRIPTION (45)
 2 0 N (44) N/A
 8310170380 831005
 PDR ADOCK 05000301
 S PDR
 NRC USE ONLY
 68 69 80

PHONE: 414/277-2811

ATTACHMENT TO LICENSEE EVENT REPORT NO. 83-011/03X-1

Wisconsin Electric Power Company
Point Beach Nuclear Plant Unit 2
Docket No. 50-301

On August 17, 1983, at 0414 hours, residual heat removal (RHR) pump 2P10B was declared out of service due to high bearing vibration levels. The out of service determination was made as a result of data obtained during monthly inservice test IT-04. This test procedure is based on ASME Code Section XI, Article IWP-3000.

This event placed the unit into a degraded mode of operation (one RHR pump available) which is permitted for 24 hours by the limiting condition for operation defined in Technical Specification 15.3.3.A.3.a, and is reportable under Technical Specification 15.6.9.2.B.2.

RHR pump 2P10B was replaced on July 1, 1983 due to seal leakage. This replacement took place during a Unit 2 refueling outage. The pump which was removed was subsequently overhauled and placed in stock as a spare.

The replacement pump was tested (per IT-04) from 1618 to 1757 hours on July 1, 1983, and showed vibration levels in the alert range based on the baseline data for the previous pump. Inservice test IT-04 was repeated on July 16, 1983 (2-week interval based on alert range data) and all bearing vibration levels were below the alert level based on baseline data for the previous pump. It should be noted that the vibration instruments were recalibrated during the 2-week interval. RHR pump 2P10B was retested per IT-04 on August 16, 1983 (one month interval based on acceptable range data) from 0359 to 0421 hours and showed horizontal vibration levels in the action-required range on both pump bearings. A retest was requested using different vibration monitors as allowed by ASME Section XI IWP-3230(d). The retest was conducted from 0334 to 0356 hours on August 17, 1983. The vibration levels were above the levels measured during the previous test and were all in the action-required range. The auxiliary operator who conducted the test stated that the pump seemed to be vibrating excessively based on past experience. At 0414 hours on August 17, 1983, pump 2P10B was declared out of service. Inservice test IT-04 was completed at 0447 hours on 2P10A to ensure that 2P10A was available while 2P10B was out of service. Maintenance personnel began work on 2P10B after 2P10A had been tested. At 1650 hours on August 17, 1983, 2P10B was released by Maintenance for testing. The repair consisted of replacing the pump with the now rebuilt pump which was removed July 1, 1983. At 2050 hours on August 17, 1983, 2P10B was declared back in service based on the successful completion of IT-04.

The cause of the high vibrations is uncertain at this time. Some bearing roughness was noted upon disassembly of the pump but it is not known if this was the sole cause of the vibrations. An evaluation is continuing to determine the cause(s) of the high vibrations.



Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

DMB

October 5, 1983

Mr. J. G. Keppler, Regional Administrator
Office of Inspection and Enforcement,
Region III
U. S. NUCLEAR REGULATORY COMMISSION
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

DOCKET NO. 50-301
LICENSEE EVENT REPORT NO. 83-011/03X-1
RESIDUAL HEAT REMOVAL SYSTEM
POINT BEACH NUCLEAR PLANT, UNIT 2

Enclosed is Licensee Event Report No. 83-011/03X-1 (an updated report) with an attachment which provides a description of an event reportable in accordance with Technical Specification 15.6.9.2.B.2, "Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation."

This Licensee Event Report was previously transmitted to you by our letter dated September 16, 1983 under LER No. 83-009/03L-0; the correct number should have been 83-011/03L-0. This updated report is being submitted to correct this error in numbering and we apologize for any inconvenience we may have caused you.

Very truly yours,

Vice President-Nuclear Power

C. W. Fay

Attachment

Copy to NRC Resident Inspector

OCT 11 1983
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