



Commonwealth Edison  
1400 Opus Place  
Downers Grove, Illinois 60515

February 27, 1995

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Attn: Document Control Desk

Subject: Additional Information Regarding Braidwood Unit 1  
Mid-Cycle Outage Plan  
NRC Docket Number 50-456

- References:
- 1) Teleconference between Commonwealth Edison Company and the Nuclear Regulatory Commission dated February 16, 1995, regarding the Mid-Cycle Inspection Plan for Braidwood Unit 1 Steam Generators
  - 2) D. Saccomando letter to Nuclear Regulatory Commission dated November 23, 1994, transmitting Braidwood Unit 1 Mid-Cycle Inspection Plan
  - 3) R. Assa letter to D. Farrar dated August 18, 1994 transmitting the Safety Evaluation for Amendment 54

Reference 3 transmitted the Nuclear Regulatory Commission (NRC)'s Safety Evaluation Report (SER) for the Use of Interim Plugging Criteria for Commonwealth Edison Company (ComEd)'s Braidwood Unit 1. In the SER, the Staff required Braidwood Station to perform a mid-cycle steam generator tube inspection on Unit 1. Reference 3 also requested that Braidwood Station submit plans for assessing the mid-cycle inspection data at least 60 days prior to the planned shutdown. Reference 2 transmitted Braidwood's plan. The referenced teleconference was held to clarify information that was submitted in the November 23rd letter. The following provides ComEd's understanding of that teleconference.

Referring to the November 23, 1994, letter:

Inspection Scope: Braidwood Station intends on conducting 100% full length inspection of all four steam generators using the bobbin coil.

Probe Wear Standard (Refer to Attachment A, page 2, Draft Generic Letter Section 3.c.4) "...since the last acceptable probe wear measurement that were within 5%, required reinspection." ComEd intends on changing the 5% to X%.

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Methodology for Braidwood Leakage Calculation For End of Cycle distribution, ComEd clarified that we would use bobbin coil voltages, apply NDE uncertainty and then do the Monte Carlo for the purpose of leakage calculation.

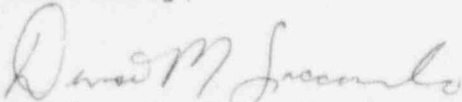
Methodology for Braidwood Conditional Probability of Burst Calculation: ComEd clarified that Braidwood will use the methodology described in WCAP 14277, "SLB Leak Rate and Tube Burst Probability Analysis Methods for ODSCC at TSP Intersections," January 1995.

Also the Staff and ComEd agreed that, "Calculation of leakage and of conditional burst probability to be performed prior to returning the S/Gs to service (mode 4) uses the as found end of midcycle voltage distribution (as opposed to a projected distribution.)"

Projected Versus Actual Inspections Results: Braidwood Station will develop projected voltage distributions for all four steam generators for the mid-cycle inspection. These projections will be compared to the actual inspection results obtained during the mid-cycle inspection. Results of this comparison will be submitted with Braidwood Station's 90 Day Inspection Report for AIM05.

If you have any questions concerning this information please contact this office.

Sincerely,



Denise M. Saccomando  
Nuclear Licensing Administrator

cc: D. Lynch, Senior Project Manager-NRR  
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