

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT
/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/9/0/3/8/3/ (8) /0/9/2/8/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On September 3 and 12, 1983, Fire Door S71-19, between the 1H Emergency Diesel /
/0/3/ / Room and the Turbine Building, would not latch. Since a fire watch was immedi- /
/0/4/ / ately posted the public health and safety were not affected. This event is /
/0/5/ / contrary to T.S. 3.7.15 and reportable pursuant to T.S. 6.9.1.9.b. /
/0/6/ / /
/0/7/ / /
/0/8/ / /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE	
/C/9/	/A/B/ (11)	/E/ (12)	/B/ (13)	/X/X/X/X/X/X/ (14)	/Z/ (15)	/Z/ (16)
	LER/RO	EVENT YEAR	SEQUENTIAL	OCCURRENCE	REPORT	REVISION
	REPORT		REPORT NO.	CODE	TYPE	NO.

(17) NUMBER /8/3/ /-/ /0/5/7/ / / /0/3/ /L/ /-/ /0/

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
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/B/ (18) /X/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /Y/ (24) /A/ (25) /C/1/7/5/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / Fire Door S71-19 would not latch because of a failed latching mechanism (Septem- /
/1/1/ / ber 3), and misalignment, (September 12). The latch was repaired, realigned and /
/1/2/ / operation of the fire door tested satisfactorily. There are no generic implica- /
/1/3/ / tions associated with this event /
/1/4/ / /

FACILITY STATUS	%POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/	/E/ (28)	/1/0/0/ (29)	/ NA / (30)	/A/ (31) / Operator Observation /

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
/1/6/	/Z/ (33)	/Z/ (34)	/ NA /

PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
/1/7/	/0/0/0/ (37)	/Z/ (38) / NA /

PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
/1/8/	/0/0/0/ (40) / NA /

LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION (43)
/1/9/	/Z/ (42) / NA /

PUBLICITY ISSUED	DESCRIPTION (45)
/2/0/	/N/ (44) / NA /

NRC USE ONLY

NAME OF PREPARER E. Wayne Harrell

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Vepco

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23117

USNRC REGION II
ATLANTA, GEORGIA

83 OCT 4 A8:52

September 28, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-136
NO/CLF: 11
Docket No. 50-338
License No. NPF-4

Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 1.

Report No.

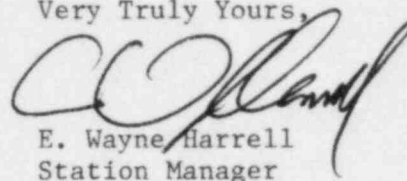
Applicable Technical Specifications

LER 83-057/03L-0

T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,



E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
016 Phillips Bldg.
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

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