

**LICENSEE EVENT REPORT**

EXHIBIT A

CONTROL BLOCK:										(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)									
<div> <div>01</div> <div>F L C R P 3</div> <div>200-000000-00</div> <div>341111</div> <div>4</div> </div>										<div> <div>78</div> <div>LICENSEE CODE</div> <div>1415</div> <div>LICENSE NUMBER</div> <div>2526</div> <div>LICENSE TYPE</div> <div>30</div> <div>57</div> <div>CAT 58</div> </div>									
<div> <div>01</div> <div>REPORT SOURCE</div> <div>6061</div> <div>050-0302</div> <div>7083083</div> <div>8092983</div> <div>9</div> </div>										<div> <div>6869</div> <div>DOCKET NUMBER</div> <div>7475</div> <div>EVENT DATE</div> <div>80</div> <div>REPORT DATE</div> </div>									
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)																			
<div> <div>02</div> <div>On August 30, 1983, while reviewing the events from the reactor trip of</div> </div>																			
<div> <div>03</div> <div>August 26, 1983, it was discovered that steam generator "A" water level</div> </div>																			
<div> <div>04</div> <div>had exceeded 360 inches (T.S. 3.4.5) between 0859 and 0906 on that date</div> </div>																			
<div> <div>05</div> <div>during a plant transient just prior to the reactor trip. After approxi-</div> </div>																			
<div> <div>06</div> <div>mately seven (7) minutes the water level was within the normal range. This</div> </div>																			
<div> <div>07</div> <div>is the first occurrence for this event, and the first report under</div> </div>																			
<div> <div>08</div> <div>T.S. 3.4.5.</div> </div>																			
<div> <div>09</div> <div> <div> <div>910</div> <div>SYSTEM CODE</div> <div>11</div> </div> <div> <div>1112</div> <div>CAUSE CODE</div> <div>13</div> </div> <div> <div>1213</div> <div>CAUSE SUBCODE</div> <div>14</div> </div> <div> <div>1415</div> <div>COMPONENT CODE</div> <div>16</div> </div> <div> <div>1516</div> <div>COMP. SUBCODE</div> <div>17</div> </div> <div> <div>1617</div> <div>VALVE SUBCODE</div> <div>18</div> </div> </div> </div>																			
<div> <div>17</div> <div> <div> <div>2122</div> <div>EVENT YEAR</div> <div>23</div> </div> <div> <div>2425</div> <div>SEQUENTIAL REPORT NO</div> <div>26</div> </div> <div> <div>27</div> <div>OCCURRENCE CODE</div> <div>28</div> </div> <div> <div>2930</div> <div>REPORT TYPE</div> <div>31</div> </div> <div> <div>32</div> <div>REVISION</div> <div>33</div> </div> </div> </div>																			
<div> <div> <div>3334</div> <div>ACTION TAKEN</div> <div>35</div> </div> <div> <div>3435</div> <div>FUTURE ACTION</div> <div>36</div> </div> <div> <div>3536</div> <div>EFFECT ON PLANT</div> <div>37</div> </div> <div> <div>3637</div> <div>SHUTDOWN METHOD</div> <div>38</div> </div> <div> <div>3738</div> <div>HOURS</div> <div>39</div> </div> <div> <div>3839</div> <div>ATTACHMENT SUBMITTED</div> <div>40</div> </div> <div> <div>3940</div> <div>NPRD-4 FORM SUB.</div> <div>41</div> </div> <div> <div>4041</div> <div>PRIME COMP. SUPPLIER</div> <div>42</div> </div> <div> <div>4142</div> <div>COMPONENT MANUFACTURER</div> <div>43</div> </div> </div>																			

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)																			
10  Main feedwater pump 2A (FWP-2A) steam control valve (ASV-1) had a mis-																			
11  aligned linkage arm which caused overfeeding to the "A" steam generator.																			
12  FWP-2A was manually tripped allowing water level to drop within limits.																			
13  At 0907 on August 26, 1983, steam generator "A" water level was below 360																			
14  inches. The linkage arm on ASV-1 was subsequently realigned.																			
15  8081  FACILITY STATUS  82  8182  % POWER  83  8283  OTHER STATUS  84  8384  METHOD OF DISCOVERY  85  8485  DISCOVERY DESCRIPTION  86																			
16  9091  ACTIVITY  92  9192  CONTENT  93  9293  RELEASED OF RELEASE  94  9394  AMOUNT OF ACTIVITY  95  9495  LOCATION OF RELEASE  96																			
17  100101  PERSONNEL EXPOSURES  102  101102  NUMBER  103  102103  TYPE  104  103104  DESCRIPTION  105																			
18  110111  PERSONNEL INJURIES  112  111112  NUMBER  113  112113  DESCRIPTION  114																			
19  120121  LOSS OF OR DAMAGE TO FACILITY  122  121122  TYPE  123  122123  DESCRIPTION  124																			
20  130131  PUBLICITY  132  131132  ISSUED  133  132133  DESCRIPTION  134																			
21  News Media Notified of Plant Trip.																			
22  140141  NAME OF PREPARER  142  141142  PHONE  143																			

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## SUPPLEMENTARY INFORMATION

REPORT NO.: 50-0302/83-035/03L-0

FACILITY: Crystal River Unit 3

REPORT DATE: September 29, 1983

OCCURRENCE DATE: August 30, 1983

### IDENTIFICATION OF OCCURRENCE:

Steam Generator "A" exceeded the 360 inch water level limit of Technical Specification 3.4.5.

### CONDITIONS PRIOR TO OCCURRENCE:

Mode I (75% full power); three pump operation.

### DESCRIPTION OF OCCURRENCE:

At 0859 on August 26, 1983, the steam control valve for Feedwater Pump 2A, ASV-I, stuck in the open position. The malfunction of ASV-I in turn caused Feedwater Pump 2A to continue to pump feedwater to Steam Generator "A". Consequently, Steam Generator "A" overfilled and exceeded the Technical Specification level limit of 360 inches for approximately 7 minutes. Subsequent level transients lead to a plant trip.

### DESIGNATION OF APPARENT CAUSE:

The steam control valve, ASV-I, stuck in the open position due to a misaligned linkage arm on the valve.

### ANALYSIS OF OCCURRENCE:

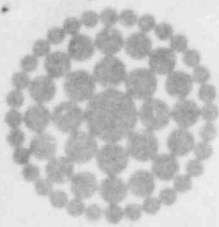
The safety significance of this event is being evaluated. This report will be updated to include the results of that evaluation.

### CORRECTIVE ACTION:

Operators manually tripped Feedwater Pump 2A. The linkage arm on ASV-I was realigned to prevent a recurrence of this event.

### FAILURE DATA:

This was the first time that the Steam Generator level limit was exceeded.



USNRC REGION II  
ATLANTA, GEORGIA

83 OCT 4 A8:52

**Florida  
Power**  
CORPORATION

September 29, 1983  
3F-0983-22

Mr. James P. O'Reilly  
Regional Administrator, Region II  
Office of Inspection & Enforcement  
U.S. Nuclear Regulatory Commission  
101 Marietta Street N.W., Suite 2900  
Atlanta, GA 30303

Subject: Crystal River Unit 3  
Docket No. 50-302  
Operating License No. DPR-72  
Licensee Event Report No. 83-035

Dear Mr. O'Reilly:

Enclosed is Licensee Event Report No. 83-035 and the attached supplementary information sheet, which are submitted in accordance with Technical Specification 6.9.1.9(b).

Should there be any questions, please contact this office.

Sincerely,

G. R. Westafer  
Manager  
Nuclear Operations Licensing and Fuel Management

AEF:feb

Enclosure

cc: Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

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