

LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	A	L	B	R	F	3	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5
7	8	9	LICENSEE CODE					14	15	LICENSE NUMBER										18	24	LICENSE TYPE				30	37	CAT 34	

CON'T

REPORT SOURCE 01 DOCKET NUMBER 6050002967 EVENT DATE 091983 REPORT DATE 092983

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

During refueling outage while performing SI 4.7.A.2.g-3 the combined leakage of the A, C, and D line MSIVs was found to be 2615.5 SCFH, 37.6 SCFH, and 2652.7 SCFH, respectively, which exceeded the allowable limit of 11.5 SCFH per valve (T.S. 4.7.A.2.i). This resulted in the total type B and C tests leakage rate exceeding 60% La (655.9 SCFH) (T.S. 4.7.A.2.g) for the A and D line MSIVs. There was no release of radioactivity as a result of MSIV leakage. There was no effect on public health or safety.

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP SUBCODE		VALVE SUBCODE					
0	9	C	D	B	E	V	A	L	V	E	X	F	D				
7	8	9	10	11	12	13	14	15	16	17	18	19	20				
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.							
17	8	3	—	0	5	4	/	0	1	T	—	0					
21	22	23	24	25	26	27	28	29	30	31	32						
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRO-1 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
B	F	2	Z	0	0	0	0	0	Y	Y	N	A	5	8	5		
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48		

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The Atwood-Morrill 26-inch globe valves were leaking past the main seat.

1 1 Leakage was caused by misalignment of poppet to main valve seat. Prior to

1 2 startup the valves will be repaired and retested until satisfactory leak

1 3 rates are obtained. A long-term MSIV improvement program is being implemented

1 4 (reference BFRO-50-260/82032).

80

FACILITY STATUS 1 5 [H] (28) % POWER 10 12 13 [0] [0] [0] (29) OTHER STATUS (30) [NA] METHOD OF DISCOVERY (31) [] DISCOVERY DESCRIPTION (32) [Surveillance tests]

ACTIVITY CONTENT 8 9 [Z] (33) 10 11 [Z] (34) 12 13 [NA] AMOUNT OF ACTIVITY (35) [] LOCATION OF RELEASE (36) [NA]

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) 1 7 [0] [0] [0] (37) [Z] (38) [NA]

PERSONNEL INJURIES NUMBER DESCRIPTION (41) 1 8 [0] [0] [0] (40) [NA]

LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43) 1 9 [Z] (42) [NA]

PUBLICITY ISSUED DESCRIPTION (45) 2 0 [N] (44) [NA]

8310060377 830929
PDR ADDCK 05000296
S PDR

IE 20

NRC USE ONLY

81

NAME OF PREPARER E. T. Holder

PHONE (205) 729-0885

LER SUPPLEMENTAL INFORMATION

BFRO-50-296 / 83054 Technical Specification Involved 3.7.A.2
4.7.A.2.1
Reported Under Technical Specification 6.7.2.a(3) * Date Due NRC 4.7.A.2.g 10/3/83

Event Narrative:

Units 1 and 3 were in a refueling outage and unit 2 was operating normally at 71% power. While performing SI 4.7.A.2.g-3 on unit 3, the combined leakage of the inboard and outboard MSIVs was found to be 2615.5 SCFH for the A line MSIVs, 37.6 SCFH for the C line MSIVs and 2652.7 SCFH for the D line MSIVs, which exceeded the allowable limit of 11.5 SCFH per valve (TS 4.7.A.2.i). Using conservative assumptions, the leak rate applicable to each valve was 1307.8 SCFH for the A line MSIVs and 1326.4 SCFH for the D line MSIVs. This resulted in the total type B and C tests leakage rate exceeding 60% La (655.9 SCFH) (TS 4.7.A.2.g) for the A and D line MSIVs. The 26-inch Atwood-Morrill globe valves were leaking past the main seat surface. This leakage was caused by misalignment of the poppet to the main valve seat. Prior to startup these valves will be repaired and retested until satisfactory leak rates are obtained. There was no release of radioactivity as a result of the MSIVs leaking. There was no effect on the health and safety of the public.

A long-term MSIV improvement program is being implemented which should improve the seating performance of these valves (reference BFRO 50-260/82032).

* Previous Similar Events:

BFRO 50-259/77023, 78034, 80003, 81014, 83020
BFRO 50-260/79007, 80092, 82030, 82037
BFRO 50-296/78025, 79014, 80058, 80059, 81073

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

September 29, 1983

83 SEP 2 09:23

83 OCT 3 09:28

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

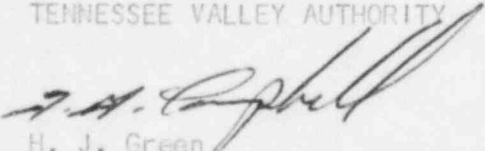
Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 3 - DOCKET
NO. 50-296 - FACILITY OPERATING LICENSE DPR-68 - REPORTABLE OCCURRENCE
REPORT BFCO-50-296/83054

The enclosed report provides details concerning main steam isolation valve
exceeding the allowable leakage rate. This report is submitted in
accordance with Browns Ferry unit 3 Technical Specification 6.7.2.a(3).

Very truly yours,

TENNESSEE VALLEY AUTHORITY


H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

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Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
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NRC Inspector, Browns Ferry

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