

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

REPORT SOURCE 60 61 DOCKET NUMBER 58 59 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

On 9-1-83 with the unit down for a planned outage, it was determined that the T.S. 3.6, limit of 0.6La(231.4SCFH) was exceeded. Valves 1B21-F010A/B, 1B21-F032A/B for feed water, valve 1B21-F016 for MS drain, 1VP036A for PC Chilled Water and valves 1E51-F068 and 1E51-F008 had excessive leakage. Valves 1B21-F065A for feedwater, 1E51-F080, 86 for RCIC vacuum breaker line, 11N074, 75 for IN penetration and 1E51-F063, 64 & 76 had leakage above the procedural guideline. In all cases there was at least one isolation valve in the piping line which had an acceptable leakage rate.

SYSTEM CODE 9 10 CAUSE CODE 11 12 CAUSE SUBCODE 12 13 COMPONENT CODE 13 14 VALVEX 14 15 COMP. SUBCODE 19 20 VALVE SUBCODE 20 21 LER/RO REPORT NUMBER 21 22 EVENT YEAR 21 22 SEQUENTIAL REPORT NO. 24 26 OCCURRENCE CODE 28 29 REPORT TYPE 30 31 REVISION NO. 32 33 ACTION TAKEN 33 34 FUTURE ACTION 34 35 EFFECT ON PLANT 35 36 SHUTDOWN METHOD 36 37 HOURS 37 40 ATTACHMENT SUBMITTED 41 42 NPRO-4 FORM SUBL 42 43 PRIME COMP. SUPPLIER 43 44 COMPONENT MANUFACTURER 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

The inboard FW check valves 1B21-F010A/B leaked when the elastomer seals were torn, the seals on the pistons for the outboard FW check valves failed. Valves 1B21-F010A/B, 32A/B, 16, 1VP063A and 1E51-F068 showed wear on the seats and were lapped. The seals on the FW check valves were replaced and 1E51-F008 had the packing repaired. The leakage for vlvs. 1B21-F065A, 1E51-F080, 86, 63, 64, 76 & 11N074/75 had acceptable leakage.

FACILITY STATUS 7 8 % POWER 10 12 OTHER STATUS 30 31 METHOD OF DISCOVERY 45 46 DISCOVERY DESCRIPTION 45 46 LOCATION OF RELEASE 45 46 PERSONNEL EXPOSURES 10 11 PERSONNEL INJURIES 11 12 LOSS OF OR DAMAGE TO FACILITY 11 12 PUBLICITY 10 11 ISSUED 10 11

8310060186 830929
PDR AD0CK 05000373
S PDR

TE22
1/1

NRC USE ONLY

NAME OF PREPARER K. C. Wittenburg

PHONE: 815-357-6761

- I. LER NUMBER: 83-107/03L-0
- II. LASALLE COUNTY STATION: Unit 1
- III. DOCKET NUMBER: 050-373
- IV. EVENT DESCRIPTION:

On September 1, 1983 with the unit shutdown for a planned outage, it was determined that the Technical Specification 3.6.1.2 limit of 0.6 La (231.4 scfh) was exceeded.

The inboard feedwater check valve, 1B21-F010B, was leaking in excess of 3900 scfh. Various valves for the feedwater penetrations, as well as for the RCIC steam lines, the main steam drain line, the RCIC turbine exhaust line and the Primary Containment "A" chilled water line also had leakage which was considered excessive. The associated valves are 1B21-F010A, 1B21-F032A, 1B21-F032B, 1B21-F016, 1VP063A, 1E51-F068 and 1E51-F008.

Several valves associated with the "A" feedwater penetration, the RCIC vacuum breaker line, and the Instrument Nitrogen line had leakage rates which exceeded the leakage guideline as stated in the procedures. These valves were 1B21-F065A, 1E51-F080, 1E51-F086, 11N074, 11N075, 1E51-F063, 1E51-F064 and 1E51-F076.

V. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

Those valves which leaked slightly in excess of the procedure guidelines, valves 1B21-F065A, 1E51-F080, 1E51-F086, 11N074, 11N075, 1E51-F063, 1E51-F064 and 1E51-F076, were considered to have acceptable leakage.

In all cases there was at least one isolation valve in the piping line which had an acceptable leakage rate. The plant was maintained in a safe condition at all times.

VI. CAUSE:

The exact cause of the inboard check valves, 1B21-F010A/B leakage has not been determined as of 25 September 1983. When the valves were opened for inspection the soft seat showed damage around the pressure relieving vent grooves, some wear on the soft seat face, and slight wear on the body seat.

The outboard feedwater check valve 1B21-F032A/B leaked because the valves would not seat properly. The valve was seated loosely when the piston gasket seals failed to seal the area around the pistons. This caused air to leak past the piston which then failed to seat the valve.

Valves 1B21-F016, 1VP063A, 1E51-F068, as well as valves 1B21-F010A, 1B21-F010B, and 1B21-F032A/B showed some wear on the valve seats which probably contributed to the excessive leakage found.

The main contributor of leakage from the four valves 1E51-F008, 1E51-F063, 1E51-F064 and 1E51-F076 which are tested together, was from the packing of the 1E51-F008 valve.

-2-

No cause for the leakage from valves 1B21-F065A, 11N074, 11N075, 1E51-F080 and 1E51-F086 has been determined as they were considered to have an acceptable leakage.

VII. CORRECTIVE ACTION:

Valves 1B21-F010A, 1B21-F010B, 1B21-F032A/B, 1B21-F016, 1VP063A and 1E51-F068 had their seats lapped. The soft seats on the 1B21-F010A/B disk were replaced by Commonwealth Edison Nuclear Engineer Department, the Station and the Anchor/Darling Valve Company. It was determined that like for like seal replacement was proper as long as any shaving on the soft seat was done by grinding under the supervision of Anchor/Darling and any burrs or cuts in the vent groove were filed to a smooth finish. A supplemental LER will be issued if additional corrective action is determined to be required. The piston gasket seals on valves 1B21-F032A/B were replaced and the packing on valve 1E51-F008 was repaired. The leakage from the rest of the valves in combination with the leakage from the repaired valves were acceptable and below the Technical Specification limit of 0.6 La. No work was therefore done on the 11N074, 11N075, 1E51-F080, and 1E51-F086 valves. The leakage from valve 1B21-F065A was accepted as it was probably leaking past manual valve 1B21-F011A. (Normally open test boundary)

Prepared by: Kermit C. Wittenburg



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

DmB

September 29, 1983

James G. Keppler
Regional Administrator
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Dear Sir:

Reportable Occurrence Report #83-107/03L-0 Docket #050-373 is being submitted to your office in accordance with LaSalle County Nuclear Power Station Technical Specification 6.6.B.2.(b), conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

G. J. Diederich
G. J. Diederich
Superintendent
LaSalle County Station

GJD/GW/rg

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U. S. NRC Document Management Branch
INPO-Records Center
File/NRC

OCT 3 1983
IE 22
11