

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Virgil C. Summer Nuclear Station										DOCKET NUMBER (2) 0 5 0 0 0 3 9 5										PAGE (3) 1 OF 1	
TITLE (4) Reactor Trip																					
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)									
01	17	84	84	005	0	02	15	84				0 5 0 0 0									
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																			
1		20.402(b)				20.406(e)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)		73.71(b)									
POWER LEVEL (10) 1 0 0		20.406(a)(1)(i)				50.38(a)(1)				<input type="checkbox"/> 50.73(a)(2)(v)		73.71(e)									
		20.406(a)(1)(ii)				50.38(a)(2)				<input type="checkbox"/> 50.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 365A)									
		20.406(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(vii)(A)											
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				<input type="checkbox"/> 50.73(a)(2)(vii)(B)											
		20.406(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																					
NAME K. W. Woodward, Manager, Operations								TELEPHONE NUMBER AREA CODE 8 0 3 3 4 5 - 5 2 0 9													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																					
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC											
A	J E			N																	
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR							
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO											
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																					
<p>On January 17, 1984, the plant tripped from 100% power because of Steam Generator "B" low level coincident with Feedwater Flow Low Reactor Trip. Surveillance Test Procedure (STP) 303.008, "Steam Generator "A" Steam Pressure (IPT-475) Instrument Operational Test," was in progress prior to the trip. The Operator at the Controls (OATC) was directed to perform step 7.1.1 which selects the Steam Flow for Steam Generator "B". The OATC selected FY-475-A but left the control channel selected to FY-484A on Steam Generator "B". As part of the STP, the I&C Technician then appropriately tripped bistables, which ultimately caused FY-484 to indicate zero steam flow for the "B" Steam Generator. The Steam Generator Water Level Control (SGWLC) System then closed the "B" Feed Reg. Valve, securing feed to the "B" Steam Generator. Actual steam flow remained near 100%. Steam Generator "B" level decreased and ultimately caused the trip. The plant response to the trip was as expected with no safety limits exceeded. The cause of this event was personnel error. The STP is being revised to clarify the required action to be performed by the OATC. This action is expected to be complete by February 15, 1984.</p>																					

8402220070 840215
PDR ADOCK 05000395
S PDR

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

February 15, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

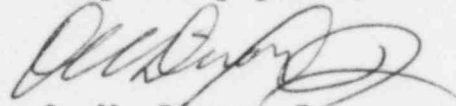
SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
LER 84-005

Dear Sir:

Please find attached Licensee Event Report #84-005 for the Virgil C. Summer Nuclear Station. This Report is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

RJB:OWD/dwf/fjc
Attachment

cc: V. C. Summer	D. A. Lavigne
T. C. Nichols, Jr./O. W. Dixon, Jr.	J. F. Heilman
E. H. Crews, Jr.	C. L. Ligon (NSRC)
E. C. Roberts	K. E. Nodland
W. A. Williams, Jr.	D. J. Richards
D. A. Nauman	C. W. Hehl
J. P. O'Reilly	J. B. Knotts, Jr.
Group Managers	INPO Records Center
O. S. Bradham	NPCF
C. A. Price	File

IE22
1/1