

Georgia Power Company
333 Piedmont Avenue
Atlanta, Georgia 30308
Telephone 404 526-6526

Mailing Address:
Post Office Box 4545
Atlanta, Georgia 30302

L. T. Gucwa
Manager Nuclear Engineering
and Chief Nuclear Engineer



February 10, 1984

Director of Nuclear Reactor Regulation
Attention: Mr. John F. Stolz, Chief
Operating Reactors Branch No. 4
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

NRC DOCKETS 50-321, 50-366
OPERATING LICENSES DPR-57, NPF-5
EDWIN I. HATCH NUCLEAR PLANT UNITS 1, 2
NUREG-0737 ITEM II.B.3, POST-ACCIDENT SAMPLING CAPABILITY

Gentlemen:

Our letter of January 26, 1984 provided information to demonstrate the extent of compliance of Plant Hatch with the criteria of the subject NUREG-0737 item. Mr. George Rivenbark, Hatch Licensing Project Manager, requested supplemental information regarding the use of sample analysis to estimate core damage during accident conditions. In response to Mr. Rivenbark's request, the attached procedure HNP-4848, "Determination of the Extent of Core Damage Under Accident Conditions," is submitted.

It should be noted that the attached procedure is subject to periodic review and revision. While changes to the procedure may be made to reflect equipment modifications, improvements in methodology, or for other reasons, the intent of the procedure will remain unchanged.

Please contact this office if there are any questions.

Very truly yours,

L. T. Gucwa

JH/mw

Enclosure

xc: (w/encl) J. T. Beckham, Jr.
H. C. Nix, Jr.
P. D. Rice
J. P. O'Reilly (NRC - Region II)
Senior Resident Inspector

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