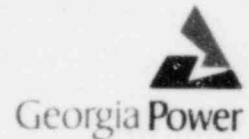


Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444



Edwin I. Hatch Nuclear Plant

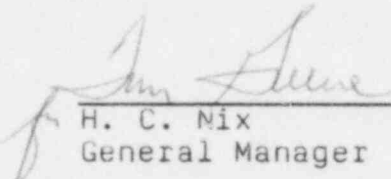
February 10, 1984
GM-84-113

PLANT E. I. HATCH
NRC Monthly Operating Report

Office of Plans and Schedules
Directorate of Licensing
United States Nuclear Regulatory Commission
Washington, D. C. 20545

Dear Sir:

Per Tech Specs section 6.9.1.6 please find attached the NRC Monthly Operating Report for Hatch Unit 1, Docket #50-321, and for Hatch Unit 2, Docket #50-366.



H. C. Nix
General Manager

HLS/hh

8402150391 840210
PDR ADOCK 05000321
R PDR

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NARRATIVE REPORT
UNIT 1

January 1st	0000	Unit at rated power.
January 6th	2356	Load reduction for weekly turbine test.
January 7th	0146	Weekly turbine test complete ramping to rated power.
January 7th	0500	Unit at rated power.
January 15th	0015	Load reduction weekly turbine test.
January 15th	0312	weekly turbine test complete ramping to rated..
January 15th	0400	Unit at rated power.
January 22nd	0440	Load reduction for weekly turbine test.
January 22nd	0950	Turbine test complete ramping to rated.
January 22nd	1400	Unit at rated power.
January 28th	0025	Load reduction for weekly turbine test.
January 28th	0150	Turbine test complete ramping to rated.
January 28th	0400	Unit at rated power.

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NARRATIVE REPORT
UNIT 2

January 1st	0000	Ramping to rated after 4160 "A" bus tripped causing "A" recirc pump to trip.
January 2nd	0000	Unit at rated power.
January 8th	0010	Load reduction for weekly turbine test.
January 8th	0500	Unit at rated power.
January 13th	0857	Ramping down for recirc pipe replacement outage & refueling.
January 13th	2009	Reactor manually scrammed to start recirc pipe replacement outage and refueling.

HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR JANUARY 1984

82-7899	11-28-83	E41, adjust u-bolt on support E41-HPCIH-35 per drawing E41-HPCIH-35 & E41-105, Ref. DCR 81-58.
83-8739	12-20-83	1E41-HPSEH-55, L31 hyd. snubbers. This snubber was leaking & was replaced with a new one.
83-8737	12-20-83	1E41-HPSEH-91B L31. This snubber was leaking it was replaced by a new one.
83-8736	12-20-83	1E41-HPSEH-92 same as above MR.
83-8734	12-20-83	1E11-RHRH-202 work same as above.
83-7903	12-02-83	1C41 pipe support adjusted u-bolt on snubber C41-F1-H001, Ref. DCR 81-58.
83-7897	12-06-83	P52 pipe support adjust u-bolt. Ref. LER 1981-103, DCR 81-58.
83-7902	11-28-83	E11 pipe support adjust u-bolt. Ref. LER 1981-103.
83-7938	12-06-83	L36 pipe insulation removed pipe insulation for u-bolt adjustment & then replaced. Ref. DCR 81-58.
83-728	12-08-83	1Z43, fabricated & install supports & installed conduits for DCR 80-348.
82-6688	10-17-83	1E11-RHRH-20 modified existing support per attached sketches for DCR 82-75.
83-7895	12-02-83	P41, adjusted u-bolts for DCR 81-58.

HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR JANUARY 1984

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
83-8491	12-08-83	1C11 reactor protection system, scram discharge volume, installed new raceways & supports Ref. DCR 82-219.
82-7603	11-11-83	1Z43, fire detection cables, disconnect cables shown on sketch B-E-80-348-275 reconnect cables as shown on sketch B-E-80-348-276 after conduit IMC-7963 is rerouted. Ref. DCR 80-348, Rev. 1.
82-7373	11-11-83	1Z43, fire detection system reworked cables for DCR 80-398.
82-5064	12-05-83	1Y42, fire protection storage tanks level switches engraved annunciator windows 79 & 80 on panel H11-655. Ref. DCR 79-462.
83-7761	12-05-83	1C11-FO14A, CRD pump discharge valve. The hand wheel on this valve was fixed so it could be turned. DCR 83-256.
83-8738	12-19-83	1E41-HPSEH-60B 3kip snubber L-31 replaced snubber with new hydraulic snubber.
83-8735	12-19-83	1E11-RHRH-249 L31 hydraulic snubbers, this snubber had a leak it was replaced with a new one.
83-8751	12-21-83	C11-LE-N060A,B,C,D & C11-LS-N660A,B,C,D reactor protection system perform DCR 82-219 modifications as per attached work package. Ref. DCR 82-219.
83-7896	12-02-83	B31, pipe supports adjust u-bolts on supports B31-101-H001 & B31-F12-H801. Ref. DCR 81-58.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR JANUARY 1984

<u>NUMBER</u>	<u>DATE COMPLETE</u>	<u>DESCRIPTION</u>
83-1858	01-09-84	2A70 ATTS pads & tubing supports, painted the ATTS pads & tubing. Ref. DCR 81-139.
83-2324	12-06-83	2C11 CRD vents & drains. Pulled cables per cable pull cards. Ref. DCR 82-206.
83-5027	11-09-83	2T28 Painted all hangers & piping on attached list. ref. DCR 82-076.
83-4943	12-06-83	2C11-N060A,B,C,D reactor protection system/scram discharge volume. Calibrated level elements & level switches. Ref. DCR 82-220.
83-4788	11-22-83	2V43 Radwaste sprinkler system engraved annunciator windows 17 & 18. Ref. DCR 82-128.
83-2428	06-08-83	2B31-G001 Elbow to pipe weld overlayed the weld with ER308L weld material as per DCR 83-63 & associated welding procedures Ref. DCR 83-63 & HNP-2-10174 & WPS-35.
83-2879	06-08-83	2B31-G001 Pipe to pipe weld overlayed the weld with ER3086 weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63 HNP-2-10174 & WSP-35.
83-2876	06-08-83	2B31-G001 pipe to elbow weld overlayed the weld with ER 308L weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174 WPS-35.
83-2877	06-08-83	2B31-G001 Pipe to pipe weld overlayed weld with weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR JANUARY 1984

83-2577	06-08-83	2B31-G001 Pipe to pipe weld overlayed weld with weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.
83-2582	06-08-83	2B31-G001 Pipe to pipe weld overlayed weld with weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.
83-2581	06-08-83	2B31-G001 Pipe to pipe weld overlayed weld with weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.
83-2583	06-08-83	2B31-G001 Pipe to elbow weld overlayed weld with ER308L weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.
83-2385	06-08-83	2B31-G001 Pipe to elbow weld overlayed weld with ER308L weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.
83-2473	06-08-83	2B31-G001 Pipe to elbow weld overlayed weld with ER308L weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.
83-2448	06-08-83	2B31-G001 Pipe to elbow weld overlayed weld with ER308L weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.
83-2522	06-08-83	2B31-G001 Pipe to elbow weld overlayed weld with ER308L weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.

83-2523	06-08-83	2B31-G001 Pipe to elbow weld overlayed weld with ER308L weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.
83-6058	11-22-83	2G31-F089 RWCU condensate pressure control valve installed 200 PSI bourdon tube in 2G31-F089. Ref. DCR 83-221.
83-2578	06-08-83	2B31-G001 Pipe to elbow weld overlayed weld with ER308L weld material as per DCR 83-63 & associated welding procedures. Ref. DCR 83-63, HNP-2-10174, WPS-35.
83-6481	12-21-83	2G31-N012 RWCU flow indicator repaired indicator. Ref. DCR 83-285.

OPERATING DATA REPORT

DOCKET NO. 50-321
 DATE 02-10-84
 COMPLETED BY: D. P. Rafeedie
 TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 1
2. Reporting Period: 01-84
3. Licensed Thermal Power (Mwt): 2436
4. Nameplate Rating (Gross MWe): 809.3
5. Design Electrical Rating (Net MWe): 777.3
6. Maximum Dependable Capacity (Gross MWe): 801.2
7. Maximum Dependable Capacity (Net MWe): 752.2
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
 The MDC (Net MWe) Value Has Been Updated
9. Power Level to Which Restricted, If Any (Net MWe)
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	744	70871
12. Number of Hours Reactor was Critical	744.0	744.0	50269.2
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	744.0	744.0	47137.0
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1799976	1799976	99018004
17. Gross Electrical Energy Generated (MWH)	599160	599160	32057330
18. Net Electrical Energy Generated (MWH)	574346	574346	30433709
19. Unit Service Factor	100.0	100.0	66.5
20. Unit Availability Factor	100.0	100.0	66.5
21. Unit Capacity Factor (Using MDC Net)	102.6	102.6	57.1
22. Unit Capacity Factor (Using DER Net)	99.3	99.3	55.2
23. Unit Forced Outage Rate	0.0	0.0	18.2
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shutdown at End of Report Period, Estimated Date of Startup:

26. Units in Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-321
 DATE: 02-10-84
 COMPLETED BY: D. P. Rafeedie
 TELEPHONE (912) 367-7851

MONTH 01-84

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
-----	--

1	767
2	774
3	774
4	774
5	773
6	775
7	756
8	775
9	773
10	766
11	773
12	777
13	776
14	776
15	765
16	773

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
-----	--

17	771
18	772
19	776
20	777
21	776
22	742
23	772
24	772
25	771
26	775
27	776
28	769
29	777
30	779
31	779

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-366
 DATE 02-10-84
 COMPLETED BY: D. P. Rafeedie
 TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 2
2. Reporting Period: 01-84
3. Licensed Thermal Power (Mwt): 2436
4. Nameplate Rating (Gross MWe): 317.0
5. Design Electrical Rating (Net MWe): 784.0
6. Maximum Dependable Capacity (Gross MWe): 803.9
7. Maximum Dependable Capacity (Net MWe): 747.9
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
 The MDC (Net MWe) Value Has Been Updated
9. Power Level to Which Restricted, If Any (Net MWe):
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	744	38497
12. Number of Hours Reactor was Critical	308.2	308.2	27379.4
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	308.2	308.2	26096.1
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	726912	726912	55943167
17. Gross Electrical Energy Generated (MWH)	242640	242640	1844420
18. Net Electrical Energy Generated (MWH)	230541	230541	17519648
19. Unit Service Factor	41.4	41.4	67.8
20. Unit Availability Factor	41.4	41.4	67.8
21. Unit Capacity Factor (Using MDC Net)	41.4	41.4	60.8
22. Unit Capacity Factor (Using DER Net)	39.5	39.5	58.0
23. Unit Forced Outage Rate	0.0	0.0	13.1
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shutdown at End of Report Period, Estimated Date of Startup:

26. Units in Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-366
 DATE: 02-10-84
 COMPLETED BY: D. P. Rafeedie
 TELEPHONE (912) 367-7851

MONTH 01-84

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
-----	--

1	647
2	768
3	775
4	776
5	780
6	779
7	780
8	771
9	778
10	773
11	773
12	777
13	524
14	-11
15	-7
16	-6

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
-----	--

17	-6
18	-6
19	-5
20	-5
21	-5
22	-5
23	-5
24	-5
25	-5
26	-3
27	-4
28	-4
29	-4
30	-4
31	-4

(3/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January

DOCKET NO. 50-321
 UNIT NAME Hatch 1
 DATE 02-10-84
 COMPLETED BY D.P. Rafeedie
 TELEPHONE 912-367-7851

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
84-1	01-06-84	S	5.1	B	5	NA	HA	TURBIN	Weekly turbine test.
84-2	01-15-84	S	2.95	B	5	NA	HA	TURBINE	Weekly turbine test.
84-3	01-22-84	S	9.34	B	5	NA	HA	TURBIN	Weekly turbine test.
84-4	01-28-84	S	3.58	B	5	NA	HA	TURBIN	Weekly turbine test.

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

⁴ Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

⁵ Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH JanuaryDOCKET NO. 50-366UNIT NAME Hatch 2DATE 02-10-84COMPLETED BY D.P. RafeedieTELEPHONE 912-367-7851

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
34-1	01-01-84	S	24	A	5	NA	CB	ELECON	Ramp from "A" 4160 bus trip which trip "A" recirc pump.
84-2	01-08-84	S	2.28	B	5	NA	HA	TURBIN	Weekly turbine test.
84-3	01-13-84	S	11.2	H	5	NA	CB	PIPEXX	Ramp down for recirc pipe replacement & refueling outage.
84-4	01-13-84	S	435.8	H	2	NA	CB	PIPEXX	Recirc pipe replacement outage.

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

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Entry Sheets for Licensee
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0161)

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