

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Millstone Point Unit 2

DOCKET NUMBER (2)

0 5 0 0 0 3 3 6 1 OF 2

TITLE (4)

Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	1	1	8	4	0	0	2	0	0	NA	0 5 0 0 0
0	1	1	8	4	0	0	2	0	0	NA	0 5 0 0 0
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)								
1			20.402(b)			20.405(e)			X 80.73(a)(2)(iv)		
POWER LEVEL (10)			30.405(a)(1)(i)			80.36(e)(1)			80.73(a)(2)(v)		
10 11 14			20.405(a)(1)(ii)			80.36(e)(2)			80.73(a)(2)(vi)		
			20.405(a)(1)(iii)			80.73(a)(2)(i)			80.73(a)(2)(vii)(A)		
			20.405(a)(1)(iv)			80.73(a)(2)(ii)			80.73(a)(2)(vii)(B)		
			20.405(a)(1)(v)			80.73(a)(2)(iii)			80.73(a)(2)(ix)		
									73.71(b)		
									73.71(e)		
									OTHER (Specify in Abstract below and in Text, NRC Form 306a)		

LICENSEE CONTACT FOR THIS LER (12)

NAME
Steve Stadnick, Plant Engineer, Ext. 4427

TELEPHONE NUMBER

AREA CODE

2 0 3 4 4 7 - 1 7 9 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

SPECIFIC ONE LINE FOR EACH COMPONENT FAILURE OCCURRING IN THE REPORTED												
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		
X	J	B	I	T	C	F	1	1	8	1	0	N

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

The plant was returning to full power operations following a refueling outage. At 15% reactor power adjustments were being made to the recently installed automatic feedwater control system to establish automatic control of the by pass valve as well as the main feedwater regulating valves.

Level in the number 1 Steam Generator started increasing and the operator took manual control to stop the increase. The increase rate was slowed, and the level gradually reached a peak of 92% before decreasing. Since level was peaking there was no need to trip the turbine, although general guidelines require a turbine trip at 90% steam generator level. Level continued to decrease, feed rate was manually increased but the added feed resulted in a level shrink due to the cold feedwater. The reactor tripped as level decreased to 38.5% finally reached 37.2%

Reactor Trip and Post Trip actions were performed in accordance with plant procedures. Reactor Trip recovery procedures were followed with personnel responding as expected.

During the transient average temperature fell to 509 degrees fahrenheit and Steam Generator pressure fell to 740 PSIA. "Normal" conditions are 530 degrees and 880 PSIA respectively. This transient occurred over a 1 hour period before the Main Steam Isolation Valves were closed to stop all heat removal. This condition is a result of early fuel cycle life and the lack of decay heat available.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Millstone Point Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 3 6 8 4 - 0 0 2 - 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

LER 84-002

The plant was returning to full power operations following an extended refueling outage. During the outage the entire Feedwater Flow Regulating System was replaced (excluding the regulating valves) with a system manufactured by the Foxboro Co. The new system was installed to improve feedwater control and improve system reliability. Since the system was new, adjustments were required to establish control parameters.

At 15% reactor power level adjustments were being made to the single element control system for the #1 Steam Generator Feedwater Control Bypass valve. The main feedwater control valve was in the closed position and adjustments had been completed on the control system for the #2 Steam Generator.

As adjustments were made to the number one (1) Steam Generator feed control equipment the water level in number one Steam Generator started increasing and the operator took manual control to stop the increase. In manual control the increase rate was slowed, and the level gradually reached a peak of 92% before it started decreasing. Since the level was peaking there was no need to trip the turbine at this point, although general guidelines require a turbine trip at 90% steam generator level. Level then continued to decrease, feed rate was manually increased but the added feed resulted in a level shrink due to the cold feedwater. The reactor tripped as level decreased to 38.5% and level finally reached a minimum value of 37.2% during this transient.

After the trip, standard post trip actions were followed by operations personnel per Operations procedure 2525. The plant and operations personnel responded as expected. Reactor trip recovery procedure 2526 was also followed. Some notable items out of "normal" range were:

Average Reactor Coolant Temperature fell below 530°F, 509°F was the lowest temperature attained.

Steam Generator Pressure fell below 880 PSIA, 740 PSIA was recorded as the low value. Both of these conditions occurred over the one hour period between actual trip and closure of the main steam isolation valves. The conditions are the result of the lack of decay heat available early in the operating cycle.

The following actions were taken to maintain reactor coolant temperature and steam generator pressure:

Steam generator blowdown flow and steam to the Steam Jet Air Ejectors were secured.

The main steam isolation valves were closed to stop all heat removal and temperature and pressure were restored to their normal operating range.

There were no Emergency Safeguards Features Actuations as a result of this transient.

The cause of the transient and subsequent reactor trip is attributable to adjustments being made to the new feedwater regulating control system. All corrective actions that could have been performed were done in accordance with operating procedures. No further action will be taken as a result of this situation.

Previous occurrences: None - not reportable under previous LER system.

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
THE HARTFORD ELECTRIC LIGHT COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
NEW YORK WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

P.O. BOX 270
HARTFORD, CONNECTICUT 06101
(203) 666-6911

February 10, 1984
MP-5779

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Reference: Facility Operating License No. DPR-65
Docket No. 50-336
Reportable Occurrence RO 50-336/84-002

Gentlemen:

This letter forwards the Licensee Event Report 84-002/3L-0 required to be submitted within thirty (30) days pursuant to paragraph 50.73 (a) (2) (IV). "Reactor Trip."

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

A handwritten signature in cursive script, appearing to read 'E. J. Mroczka'.

E. J. Mroczka
Station Superintendent
Millstone Nuclear Power Station

EJM/SS:ejl

Attachment: LER 50-336//84-002

cc: Dr. T. E. Murley, Region I

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