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Vice President and General Manager
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Georgia Power

the southern electric system

NED-83-648

February 6, 1984

Director of Nuclear Reactor Regulation
Attention: Mr. John F. Stolz, Chief
Operating Reactors Branch No. 4
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

NRC DOCKET 50-321
OPERATING LICENSE DPR-57
EDWIN I. HATCH NUCLEAR PLANT UNIT 1
SUBMITTAL OF HPCI AND RCIC CHANGE PROPOSAL

Gentlemen:

In accordance with the provisions of 10 CFR 50.90 as required by the provisions of 10 CFR 50.59(c)(1), Georgia Power Company (GPC) hereby proposes an amendment to the Edwin I. Hatch Unit 1 Technical Specifications (Appendix A to the Operating License). This amendment would change the requirements for the High Pressure Coolant Injection (HPCI) system and the Reactor Core Isolation Cooling (RCIC) system to require the systems to be operable when the reactor vessel pressure is greater than 150 psig.

GPC has had a review performed by our NSSS and fuel supplier, General Electric Co., which shows that this change would have no effect on the Plant Hatch Unit 1 FSAR analyses of peak fuel cladding temperature changes during analyzed accidents and transients using NRC approved methodology.

The PRB and the SRB have reviewed and determined that application of these proposed Technical Specification changes would not constitute an unreviewed safety question. The probability of occurrence or the consequences of an accident or malfunction of safety-related equipment would not be increased above those analyzed in the FSAR, because the Hatch Unit 1 FSAR accident/transient analyses and performance specifications for HPCI and RCIC do not take credit for operation of HPCI or RCIC below a reactor pressure of 150 psig. The possibility of an accident or malfunction different from those analyzed in the FSAR would not result from these changes, since these systems would not be operated in a manner new or different from that described in the FSAR. The margin of safety as analyzed in Technical Specifications would not be reduced because the Plant Hatch

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Unit 1 Technical Specifications do not require surveillance testing of HPCI or RCIC below a reactor pressure of 150 psig. Furthermore, the Low Pressure Coolant Injection System, Automatic Depressurization System, and Core Spray System, are all still required to be operational in the range of reactor pressure between 113 and 150 psig, as is consistent with the FSAR analyses.

Instructions for incorporation of these changes (Attachment 1) along with copies of affected Technical Specification pages are enclosed.

Included with this proposal is a determination of amendment class (Attachment 2). We have determined this to be one Class III amendment, and have enclosed the appropriate payment.

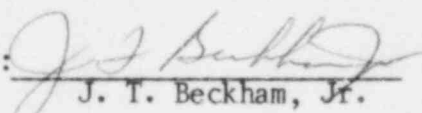
In accordance with the requirements of 10 CFR 50.92, a Significant Hazards Review for each change to the Technical Specifications is enclosed (Attachment 3).

Included with this submittal is a safety evaluation, performed by General Electric Company, of the the proposed changes entitled, "HPCI/RCIC Operability Limit, Edwin I. Hatch Nuclear Power Plant - Unit 1."

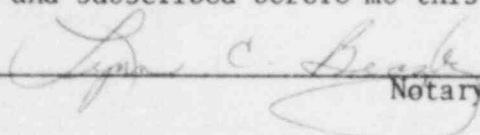
Pursuant to the requirements of 10 CFR 50.92, J. L. Ledbetter of the Georgia Department of Natural Resources will be sent a copy of this letter and all applicable attachments.

J. T. Beckham, Jr. states that he is Vice President of Georgia Power Company and is authorized to execute this oath on behalf of Georgia Power Company, and that to the best of his knowledge and belief the facts set forth in this letter are true.

GEORGIA POWER COMPANY

By: 
J. T. Beckham, Jr.

Sworn to and subscribed before me this 6th day of February, 1984.


Notary Public

CBS
Enclosure

xc: H. C. Nix, Jr.
Senior Resident Inspector
J. P. O'Reilly, (NRC-Region II)
J. L. Ledbetter

Notary Public, Georgia, State at Large
My Commission Expires July 26, 1985

ATTACHMENT 1

NRC DOCKET 50-321
OPERATING LICENSE DPR-57
EDWIN I. HATCH NUCLEAR PLANT UNIT 1
SUBMITTAL OF HPCI AND RCIC CHANGE PROPOSAL

The proposed changes to Technical Specifications (Appendix A to Operating License DPR-57) would be incorporated as follows:

<u>Remove page</u>	<u>Insert Page</u>
3.5-6	3.5-6
3.5-7	3.5-7
3.5-8	3.5-8
3.5-16	3.5-16
3.5-17	3.5-17

ATTACHMENT 3

NRC DOCKET 50-321
OPERATING LICENSE DPR-57
EDWIN I. HATCH NUCLEAR PLANT UNIT 1
SUBMITTAL OF HPCI AND RCIC CHANGE PROPOSAL

1. Change "113 psig" to "150 psig" in Sections 3.5.D.1.a.(2), 3.5.D.3, 3.5.E.1.a.(2), and 3.5.E.3:

BASIS:

These changes would make the Technical Specification requirements consistent with the modes of HPCI and RCIC operation described in the FSAR, and would remove the present Technical Specification inconsistency between Plant Hatch Units 1 and 2 with regard to the HPCI/RCIC minimum reactor pressure operability limit. Reviews performed in accordance with NRC approved methodology show that these changes would have no effect on the FSAR analyses of peak fuel cladding temperature changes during analyzed accidents and transients. Therefore, these changes would not result in any increase in the probability or consequences of a postulated accident, or a decrease in the margin of safety. No unanalyzed type of accident or transient would result from these changes. Furthermore, these changes are clearly within all acceptable criteria. Therefore, these changes are consistent with Item (vi) of the "Examples of Amendments that are Considered Not Likely to Involve Significant Hazards Considerations" listed on page 14,870 of the April 6, 1983, issue of the Federal Register.

2. Change the Bases to reflect the above changes:

BASIS:

These changes would make the Technical Specification requirements consistent with the modes of HPCI and RCIC operation described in the FSAR, and would remove the present Technical Specification inconsistency between Plant Hatch Units 1 and 2 with regard to the HPCI/RCIC minimum reactor pressure operability limit. Reviews performed in accordance with NRC approved methodology show that these changes would have no effect on the FSAR analyses of peak fuel cladding temperature changes during analyzed accidents and transients. Therefore, these changes would not result in any increase in the probability or consequences of a postulated accident, or a decrease in the margin of safety. No unanalyzed type of accident or transient would result from these changes. Furthermore, these changes are clearly within all acceptable criteria. Therefore, these changes are consistent with Item (vi) of the "Examples of Amendments that are Considered Not Likely to Involve Significant Hazards Considerations" listed on page 14,870 of the April 6, 1983, issue of the Federal Register.