

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

00 017-926

Facility: BSEP Unit No. 2

Event Date: February 10, 1983

While performing a routine inspection in the Unit No. 2 drywell, a resident NRC inspector observed that the instrument air tubing supplying the accumulators of the unit's SRV/ADS valves appeared to lack adequate support. An engineering inspection and evaluation of the subject tubing determined the tubing support was not in accordance with the plant design requirements.

The most probable cause of this deficiency is attributed to rerouting of the tubing during the installation of a plant modification which installed two-stage SRVs to replace the former three-stage design SRVs. Rerouting of the subject tubing occurred without adequate procedural controls in the modification installation package. This resulted from an oversight on the part of the responsible engineer, who did not realize the design requirement that the tubing be supported in accordance with the spacing table requirements of ANSI B31.1.1

Additional supports were installed on the subject tubing in accordance with Plant Modification 83-25 to ensure structural integrity of the tubing during a seismic event. Additional supports will be added during a future unit outage in order to bring the tubing system into full compliance with design requirements.

The respective Unit No. 1 SRV/ADS valve accumulator air tubing was inspected and modified as necessary during the 1983 Unit No. 1 refueling outage in order to allow full support design compliance prior to the subsequent startup of the unit. In addition, prior to startup of Unit No. 1, a comprehensive inspection program culminating in seismic simulation/tubing integrity testing was performed on Unit No. 1 to ensure compliance with structural integrity requirements.

As a result of this event, a plant memorandum was distributed on February 22, 1983, to all Engineering personnel, describing this event and outlining the requirements for plant instrument air tubing support. In lieu of the training program addressed in our original response, our modification procedure will be revised to add additional controls to insure design bases are completely defined. This revision will be completed by June 30, 1984.



Carolina Power & Light Company

Brunswick Steam Electric Plant  
P. O. Box 10429  
Southport, NC 28461-0429

January 30, 1984

FILE: B09-13510C  
SERIAL: BSEP/83-3531

Mr. James P. O'Reilly, Administrator  
U. S. Nuclear Regulatory Commission  
Region II, Suite 3100  
101 Marietta Street N.W.  
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-324  
LICENSE NO. DPR-62  
SUPPLEMENT TO LICENSEE EVENT REPORT 2-83-19

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.8i of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 2, the enclosed supplemental Licensee Event Report is submitted. The original report fulfilled the requirement for a written report within fourteen (14) days of a reportable occurrence and both are in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

C. R. Dietz, General Manager  
Brunswick Steam Electric Plant

RMP/da

Enclosure

cc: Mr. R. C. DeYoung  
NRC Document Control Desk

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