

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 7										PAGE (3) 1 OF 0 2																																		
TITLE (4) Reactor Trip																																																						
EVENT DATE (5)									LER NUMBER (6)									REPORT DATE (7)									OTHER FACILITIES INVOLVED (8)																											
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES												DOCKET NUMBER(S)															
0 1			1 0			8 4			8 4			0 0			6 0			0 0			0 2			0 8			8 4															0 5 0 0 0												
0 1			1 0			8 4			8 4			0 0			6 0			0 0			0 2			0 8			8 4															0 5 0 0 0												
OPERATING MODE (9) 1										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)																																												
POWER LEVEL (10) 0 1 1 3										20.402(b)										20.406(c)										<input checked="" type="checkbox"/> 50.73(a)(2)(iv)										73.71(b)														
										20.406(a)(1)(i)										50.36(c)(1)										<input type="checkbox"/> 50.73(a)(2)(v)										73.71(c)														
										20.406(a)(1)(ii)										50.36(c)(2)										<input type="checkbox"/> 50.73(a)(2)(vii)										OTHER (Specify in Abstract below and in Text, NRC Form 366A)														
										20.406(a)(1)(iii)										50.73(a)(2)(ii)										<input type="checkbox"/> 50.73(a)(2)(viii)(A)																								
										20.406(a)(1)(iv)										50.73(a)(2)(iii)										<input type="checkbox"/> 50.73(a)(2)(viii)(B)																								
20.406(a)(1)(v)										50.73(a)(2)(iii)										<input type="checkbox"/> 50.73(a)(2)(ix)																																		
LICENSEE CONTACT FOR THIS LER (12)																																																						
NAME Glenn B. Kirk, Compliance Section Engineer																				TELEPHONE NUMBER AREA CODE 6 1 5 8 7 0 - 6 1 4 6																																		
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																						
CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NPRDS					CAUSE					SYSTEM					COMPONENT					MANUFACTURER					REPORTABLE TO NPRDS									
SUPPLEMENTAL REPORT EXPECTED (14)																														EXPECTED SUBMISSION DATE (15)										MONTH DAY YEAR														
YES (If yes, complete EXPECTED SUBMISSION DATE)																				<input checked="" type="checkbox"/> NO																																		

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0204 CST on 01/10/84, unit 1 experienced a turbine and reactor trip. Unit 1 was in mode 1 (2230 psig, 552 degrees F) at 13% reactor power just prior to the event. The turbine trip was caused by high-high level in the number four steam generator. Subsequent steam generator level shrink resulted in a reactor trip due to low-low level in the number two steam generator. Unit 1 stabilized at 547 degrees F following the reactor trip.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Sequoyah, Unit 1	0 5 0 0 0 3 2 7	8 4	0 0 6	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 368A's) (17)

During a normal startup following a reactor shutdown on 01/09/84 (see SQRO-50-327/84005) unit 1 experienced a reactor trip. Just prior to the event which occurred at 0204 CST on 01/10/84, unit 1 was in mode 1 (2230 psig, 552 degrees F) at 13% reactor power with steamflow and feedflow at approximately 10% and pressurizer level at 40%.

When reactor coolant system pressure began to decrease slightly, the reactor operator increased reactor power to compensate. This reactor power increase apparently caused the steam dumps, which were in pressure control, to momentarily open. This opening of the steam dump valves resulted in swelling of steam generator levels. This action resulted in a high-high level in the number four steam generator which caused a turbine trip and main feedwater isolation to all four steam generators. The transient then resulted in shrink of steam generator levels causing a reactor trip on low-low level in steam generator number two. Investigation revealed that the decrease in reactor coolant system pressure was due to failure of one bank of pressurizer backup heaters. The heaters had failed to energize due to burned contacts which prevented the breaker from making proper contact. The cause of the burned contacts was due to an earlier failure of the heater breaker linkage which had caused the trip coil to short out. The breaker had been replaced prior to this reactor startup, but maintenance had failed to realize the damage to the contacts.

The backup heater breaker contacts were replaced and the breaker reinstalled. Additional evaluation indicates that sluggish response of the steam generator feedwater regulator valves during the feedwater isolation may have contributed to the event. The feedwater regulator valves were lubricated and stroked to improve response. All equipment and personnel responded and performed as expected following the reactor trip.

There was no effect on public health or safety. Unit 1 stabilized at 547 degrees F following the reactor trip.

Previous occurrences - none.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

February 8, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555


Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.
50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT
SQRO-50-327/84006

The enclosed licensee event report provides details concerning a reactor trip from 13% reactor power. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



C. C. Mason
Power Plant Superintendent

Enclosure
cc (Enclosure):

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NRC Inspector, NUC PR, Sequoyah

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Certified By Ray L. Lott