

Commonwealth Edison Company
LaSalle Generating Station
2601 North 21st Road
Marseilles, IL 61341-9757
Tel 815-357-6761



February 10, 1995

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for January, 1995.

A handwritten signature in dark ink, appearing to read "D. J. Ray".

D. J. Ray
Station Manager
LaSalle County Station

DJR/tmb

Enclosure

cc: John B. Martin, Regional Administrator - Region III
NRC Senior Resident Inspector - LaSalle
IL Department of Nuclear Safety - LaSalle
IL Department of Nuclear Safety - Springfield, IL
NRR Project Manager - Washington, D.C.
GE Representative - LaSalle
Regulatory Assurance Supervisor - LaSalle
Licensing Operations Director - Downers Grove
Nuclear Fuel Services Manager - General Office
Off-Site Safety Review Senior Participant - Downers Grove
INPO Records Center
Central File

110239

9502140383 950131
PDR ADOCK 05000373
R PDR

A Unicom Company

JE241

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

January 1995

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

TABLE OF CONTENTS
(UNIT 1)

I. INTRODUCTION

II. REPORT

A. SUMMARY OF OPERATING EXPERIENCE

B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

C. LICENSEE EVENT REPORTS

D. DATA TABULATIONS

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

E. UNIQUE REPORTING REQUIREMENTS

1. Main Steam Safety Relief Valve Operations
2. Major Changes to Radioactive Waste Treatment System
3. Static O-Ring Failures
4. Off-Site Dose Calculation Manual Changes

I. INTRODUCTION (UNIT 1)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1130 Mwe.
4	0300	Reduced power level to 1000 Mwe to perform a rod set.
	1000	Increased power level to 1145 Mwe.
12	2300	Reduced power level to 830 Mwe to perform a rod set.
13	0500	Increased power level to 1120 Mwe.
	0600	Reduced power level to 990 Mwe to perform a rod set.
	1100	Increased power level to 1100 Mwe.
21	0500	Reduced power level to 1070 Mwe to changeover Condensate/Condensate Booster pumps.
	0900	Increased power level to 1135 Mwe.
31	2400	Reactor critical, Generator on-line at 1140 Mwe.

AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION
(None)

C. LICENSEE EVENT REPORTS (Unit 1)

<u>LER No.</u>	<u>Date</u>	<u>Description</u>
95-001	01/06/95	Instrument stop valve for Residual Heat Removal (RHR) Pump 1A high pressure switch found closed. This instrument provides for an alarm in the Control Room.
95-003	01/27/95	TIP Ball Valves will re-open upon reset of a Group 7 isolation if the valve control switch is left in the open position.
95-004	01/31/95	Determination that a Senior Reactor Operator was absent from the Control Room for 3 minutes 32 seconds on May 26, 1994.

D. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 1)
2. Average Daily Unit Power Level (See Table 2)
3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 1)

1. Safety Relief Valve Operations
(None)
2. Major Changes to Radioactive Waste Treatment Systems
(None)
3. Static O-Ring Failures
(None)
4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE February 10, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD:	January 1995	GROSS HOURS IN REPORTING PERIOD	744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323	MAX DEPEND CAPACITY (MWe-Net):	1,036
		DESIGN ELECTRICAL RATING (MWe-N	1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASONS FOR RESTRICTION (IF ANY):

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	744.0	67,383.9
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	744.0	744.0	65,855.2
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWht)	2,457,885	2,457,885	195,144,710
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	839,182	839,182	65,230,073
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	813,809	813,809	62,582,953
12. REACTOR SERVICE FACTOR (%)	100.0	100.0	69.3
13. REACTOR AVAILABILITY FACTOR (%)	100.0	100.0	71.0
14. UNIT SERVICE FACTOR (%)	100.0	100.0	67.8
15. UNIT AVAILABILITY FACTOR (%)	100.0	100.0	67.8
16. UNIT CAPACITY FACTOR (USING MDC) (%)	105.6	105.6	62.1
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	101.5	101.5	59.7
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	8.2

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE February 10, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: January 1995

DAY	POWER	DAY	POWER
1	1,081	17	1,095
2	1,088	18	1,098
3	1,076	19	1,097
4	1,087	20	1,097
5	1,099	21	1,093
6	1,097	22	1,099
7	1,098	23	1,098
8	1,098	24	1,099
9	1,098	25	1,098
10	1,096	26	1,098
11	1,099	27	1,099
12	1,096	28	1,099
13	1,042	29	1,100
14	1,094	30	1,101
15	1,094	31	1,100
16	1,095		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)
--------------------------------	------------------	-----------------------------------	---------------------	--------	--	--

(None)

SUMMARY OF OPERATION:

The unit remained on-line at high power throughout the month. Several minor power reductions were required due to maintenance and surveillance activities.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

January 1995

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

TABLE OF CONTENTS
(UNIT 2)

I. INTRODUCTION

II. REPORT

A. SUMMARY OF OPERATING EXPERIENCE

B. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

C. LICENSEE EVENT REPORTS

D. DATA TABULATIONS

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

E. UNIQUE REPORTING REQUIREMENTS

1. Main Steam Safety Relief Valve Operations
2. Major Changes to Radioactive Waste Treatment System
3. Static O-Ring Failures
4. Off-Site Dose Calculation Manual Changes

I. INTRODUCTION (UNIT 2)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit two was issued operating license number NFF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1130 Mwe.
	0300	Reduced power to 930 Mwe due to system load requirements.
2	0400	Increased power level to 1140 Mwe.
5	0300	Reduced power level to 1000 Mwe to perform a rod set.
	0600	Increased power level to 1140 Mwe.
9	0100	Reduced power level to 1100 Mwe for control rod drive hydraulic control unit accumulator replacements.
10	0100	Reduced power level to 900 Mwe to perform scram time testing.
	1000	Increased power level to 1135 Mwe.
11	0000	Reduced power level to 830 Mwe to perform scram time testing.
	2200	Increased power level to 1130 Mwe.
12	0200	Reduced power level to 900 Mwe to perform scram time testing.
	1000	Increased power level to 1140 Mwe.
13	0300	Reduced power level to 900 Mwe to perform a rod set and scram time testing.
	0900	Increased power level to 1140 Mwe.
16	2000	Reduced power level to 1100 Mwe due to changeover the Feedwater Heater Drain pumps.
	2400	Increased power level to 1140 Mwe.
18	0200	Reduced power level to 850 Mwe due to changeover the Condensate/Condensate Booster pumps.
	0900	Increased power level to 1140 Mwe.
22	0000	Reduced power level to 1100 Mwe due to changeover the Feedwater Heater Drain pumps.
	0600	Increased power level to 1140 Mwe.
25	2105	Reduced power level to 750 Mwe for surveillance performance and to perform a rod set.
26	0800	Increased power level to 1140 Mwe.
31	2400	Reactor critical, Generator on-line at 1140 Mwe.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION
(None)

C. LICENSEE EVENT REPORTS (Unit 2)

<u>LER No.</u>	<u>Date</u>	<u>Description</u>
95-001	01/12/95	During performance of instrumentation surveillance testing a hydraulic transient occurred in a sensing line, resulting in spurious actuation of Division I Low Pressure ECCS and RCIC. ECCS did not inject into the reactor due to reactor pressure being above the low pressure injection permissive. RCIC injected but no reactor water level change was observed.

D. DATA TABULATIONS (Unit 2)

1. Operating Data Report (See Table 1)
2. Average Daily Unit Power Level (See Table 2)
3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 2)

1. Safety Relief Valve Operations
(None)
2. Major Changes to Radioactive Waste Treatment Systems
(None)
3. Static O-Ring Failures
(None)
4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE February 10, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: January 1995 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASONS FOR RESTRICTION (IF ANY):

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	744.0	65,952.0
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	744.0	744.0	64,729.5
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWht)	2,425,194	2,425,194	196,177,122
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	830,597	830,597	65,519,166
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	805,795	805,795	62,990,557
12. REACTOR SERVICE FACTOR (%)	100.0	100.0	73.1
13. REACTOR AVAILABILITY FACTOR (%)	100.0	100.0	75.0
14. UNIT SERVICE FACTOR (%)	100.0	100.0	71.8
15. UNIT AVAILABILITY FACTOR (%)	100.0	100.0	71.8
16. UNIT CAPACITY FACTOR (USING MDC) (%)	104.5	104.5	67.4
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	100.5	100.5	64.8
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	10.5

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

Refuel Outage (L2R06), 02/18/95, 8 weeks

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE February 10, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: January 1995

DAY	POWER	DAY	POWER
1	956	17	1,102
2	1,074	18	1,077
3	1,100	19	1,097
4	1,094	20	1,099
5	1,091	21	1,101
6	1,104	22	1,097
7	1,102	23	1,100
8	1,104	24	1,098
9	1,076	25	1,072
10	1,057	26	1,001
11	1,037	27	1,097
12	1,068	28	1,099
13	1,072	29	1,102
14	1,099	30	1,101
15	1,099	31	1,102
16	1,097		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 2)

YEARLY SEQUENTIAL NUMBER	METHOD OF DATE (YYMMDD)	CORRECTIVE TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	SHUTTING DOWN THE REACTOR OR REDUCING POWER	ACTIONS/COMMENTS (LER # if applicable)
(None)						

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system load, maintenance and surveillance activities.