

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 7										PAGE (3) 1 OF 0 2			
TITLE (4) Reactor Trip																							
EVENT DATE (5)				LER NUMBER (6)				REPORT DATE (7)				OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES						DOCKET NUMBER(S)								
0	1	1	0	8	4	0	0	7							0 5 0 0 0								
																		0 5 0 0 0					
OPERATING MODE (9) 1				THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																			
POWER LEVEL (10) 0 2 1 3				20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)							
				20.406(a)(1)(i)				50.36(e)(1)				50.73(a)(2)(v)				73.71(e)							
				20.406(a)(1)(ii)				50.36(e)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)							
				20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)											
				20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)											
				20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)											
LICENSEE CONTACT FOR THIS LER (12) Glenn B. Kirk, Compliance Section Engineer																							
NAME																		TELEPHONE NUMBER					
																		AREA CODE					
																		6 1 5 8 7 0 - 6 1 4 6					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																							
CAUSE	SYSTEM	COMPONENT	MANUFAC- Turer	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFAC- Turer	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFAC- Turer	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFAC- Turer	REPORTABLE TO NRC				
SUPPLEMENTAL REPORT EXPECTED (14)																		EXPECTED SUBMISSION DATE (15)					
YES (If yes, complete EXPECTED SUBMISSION DATE)																		MONTH DAY YEAR					
<input checked="" type="checkbox"/> NO																							

At 1610 CST on 01/10/84, unit 1 experienced a turbine and reactor trip. Unit 1 was in mode 1 (2235 psig, 557 degrees F) at 23% reactor power just prior to the event. The turbine trip was caused by a high-high level in the number three steam generator. Subsequent steam generator level shrink resulted in a reactor trip due to low-low level in the number four steam generator. Unit 1 stabilized at 547 degrees F following the reactor trip.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOC#	NUMBER (2)	LER NUMBER (6)			PAGE (3)		
			YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Sequoyah, Unit 1		0 5 0 0 0 3 7	8 4	0 0 7	0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During normal startup following recovery from a reactor trip (see SQRO-50-327/84006), unit 1 experienced another reactor trip. Just prior to the event which occurred at 1610 CST on 01/10/84, unit 1 was in mode 1 (2235 psig, 557 degrees F) at 23% reactor power. The balance of plant operator was controlling steam generator levels with the main feedwater regulator valves in manual control. The operator had shut the feedwater regulator bypass valves in preparation to place the steam generator level controls in automatic. With the bypass valves closed, the operator only has "course" control of the steam generator levels. This reactor trip has been attributed to operator error in that the bypass valves were closed too early. Without the "fine" control of steam generator levels by use of the bypass valves, the operator allowed the level in steam generator number three to go high. A turbine trip and feedwater isolation resulted from high-high level in the number three steam generator. The reactor operator immediately reduced reactor power in an attempt to prevent a reactor trip. With reactor power at 4%, the steam generator level shrink due to the feedwater isolation resulted in low-low level in the number four generator which tripped the reactor. The unit stabilized at 547 degrees F following the reactor trip.

There was no effect on public health or safety. All equipment and personnel performed and responded as expected following the reactor trip.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy Daisy, Tennessee 37379

February 8, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.
50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT
SQRO-50-327/84007

The enclosed licensee event report provides details concerning a reactor trip from 4% reactor power. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

C. C. Mason

C. C. Mason
Power Plant Superintendent

Enclosure
cc (Enclosure):

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Suite 1500
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NRC Inspector, NUC PR, Sequoyah

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