

CONTROL BLOCK: ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ ☐ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)01 C A S I O S 2 2 0 0 0 - 0 0 0 0 0 - 0 0 0 3 4 1 1 1 1 4 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

CONT

01 REPORT SOURCE L 6 0 5 0 0 0 3 6 1 7 1 2 2 2 8 3 8 0 1 2 4 8 4 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100
REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On 12/22/83, at 1055, with Unit 2 in Mode 1, an inspection of Snubber

03 S2-ST-017-H-009 located on the main steam supply line to the Auxiliary

04 Feed Pump Turbine K007 determined it to be binding in the compression

05 mode. The snubber was replaced in accordance with LCO 3.7.6 on

06 12/24/83. However, the required engineering evaluation for the steam

07 supply line was not completed by 1055 on 12/25/83 as required by LCO

08 3.7.6. (See Attachment)

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
Z Z 11 D 12 Z 13 Z Z Z Z Z Z 14 Z 15 Z 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

17 LER/RO REPORT NUMBER EVENT YEAR
8 3
21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

ACTION FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
X 18 G 19 Z 20 Z 21 0 0 0 0 0 Y 23 N 24 Z 25 Z 9 9 9 9
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Failure to perform the required engineering evaluation was due to not

11 having adequate administrative controls to document, track, and assign

12 responsibility for engineering evaluations on components attached to

13 inoperable snubbers. Engineering Procedure S0123-V-5.1 has been

14 appropriately revised. Snubber failure is currently under evaluation.

15 FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION
E 28 1 0 0 29 NA B 31 Records Review
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE
Z 33 Z 34 NA NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION
0 0 0 37 Z 38 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

18 PERSONNEL INJURIES NUMBER DESCRIPTION
0 0 0 40 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION
Z 42 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

20 PUBLICITY ISSUED DESCRIPTION
N 44 NA
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

NAME OF PREPARER

J. G. HAYNES

PHONE: 714-492-7700

ATTACHMENT TO LER 83-156
SOUTHERN CALIFORNIA EDISON COMPANY
SAN ONOFRE NUCLEAR GENERATING STATION
UNIT NO. 2, DOCKET NO. 50-361

SUPPLEMENTAL INFORMATION FOR EVENT DESCRIPTION AND PROBABLE
CONSEQUENCES

The omitted engineering evaluation was discovered on January 11, 1984, and action was initiated to place the unit in Mode 3 within 6 hours in accordance with LCO 3.0.3. The required engineering evaluation on the steam supply line was completed and LCO 3.0.3 was exited at 1722 on January 11, 1984. There was no impact on the health and safety of plant personnel or the public associated with this event.

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

J. G. HAYNES
STATION MANAGER

January 24, 1984

SCE

TELEPHONE
(714) 492-1700

1984 JAN 27 PM 12 15

RECEIVED

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 83-156
San Onofre Nuclear Generating Station, Unit 2

Pursuant to Section 6.9.1.13.b of Appendix A, Technical Specifications to Facility Operating License NPF-10 for San Onofre Unit 2, this submittal provides the required 30-day written report and a copy of the Licensee Event Report (LER) form for an occurrence involving Limiting Conditions for Operation (LCO's) 3.7.1.2 and 3.7.6 associated with the Auxiliary Feedwater System and snubbers, respectively. This report was delayed in order to provide a complete response.

On December 22, 1983, at 1055, with Unit 2 in Mode 1 at 100% power, during an inspection of the seismic snubbers in accordance with Maintenance Procedure SO23-I-2.29 for the main steam supply line to the Auxiliary Feed Pump Turbine (AFWT) K007, Snubber S2-ST-017-H-009 was found to be binding.

The Action Statement associated with LCO 3.7.6 requires that within 72 hours of the discovery of an inoperable snubber, the snubber must be replaced or restored to operable status and an engineering evaluation must be completed per Technical Specification 4.7.6.g on the attached component. The snubber was replaced and declared operable at 0300 on December 24, 1983. However, the engineering evaluation was not completed within the time limit of the Action Statement.

IE-22 11

January 24, 1984

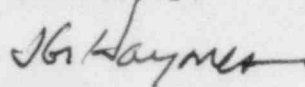
On January 11, 1984, the failure to perform this evaluation was discovered during the review of the December 22 event by Station Engineering. At 1545 with Unit 2 in Mode 1 at 100% power, action was initiated to place Unit 2 in Mode 3 within 6 hours in accordance with LCO 3.0.3. By 1702, the engineering evaluation on the steam supply line to AFWT K007 was completed, it concluded that the operability of the attached system was not impacted by the inoperable snubber, and the plant shutdown was halted. This evaluation arrived in the Control Room at 1722 and LCO 3.0.3 was exited.

The cause of the snubber failure is currently under evaluation by SCE and the vendor, Pacific Scientific. Failure to perform the engineering evaluation in a timely manner is attributed to not having adequate administrative controls to document, track and assign responsibility for performing the Technical Specification 4.7.6.g engineering evaluation. As corrective action to prevent recurrence, Engineering Procedure S0123-V-5.1, "Nonconforming Material, Parts or Components" has been appropriately revised.

There was no impact on the health and safety of plant personnel or the public associated with this event.

If you require any additional information, please so advise.

Sincerely,



Enclosure: LER No. 83-156

January 24, 1984

cc: A. E. Chaffee (USNRC Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

U. S. Nuclear Regulatory Commission
Division of Technical Information and Document Control

Institute of Nuclear Power Operations (INPO)