



**Boston Edison**

Pilgrim Nuclear Power Station  
Rocky Hill Road  
Plymouth, Massachusetts 02360

**L. J. Olivier**

Vice President Nuclear Operations  
and Station Director

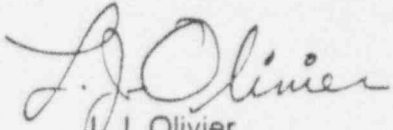
January 13, 1995  
BECO Ltr. #95-003

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Docket No. 50-293  
License No. DPR-35

DECEMBER 1994 MONTHLY REPORT

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

  
L.J. Olivier

WJM/lam/9458

Attachment

cc: Mr. Thomas T. Martin  
Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Senior Resident Inspector

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293  
DATE: 1/13/95  
COMPLETED BY: W. Munro  
TELEPHONE: (508) 830-8474

MONTH December 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	62	17	609
2	85	18	414
3	68	19	641
4	485	20	664
5	656	21	666
6	594	22	665
7	645	23	666
8	664	24	665
9	659	25	665
10	664	26	664
11	663	27	665
12	665	28	665
13	666	29	664
14	666	30	665
15	665	31	665
16	665		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

# OPERATING DATA REPORT

DOCKET NO. 50-293  
 DATE 1/13/95  
 COMPLETED BY: W. Munro  
 TELEPHONE (508) 830-8474

## OPERATING STATUS

## NOTES

1. Unit Name Pilgrim I
2. Reporting Period December 1994
3. Licensed Thermal Power (MWt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:  
NONE

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>744.0</u>	<u>8760.0</u>	<u>193392.0</u>
12. Hours Reactor Critical	<u>744.0</u>	<u>6288.6</u>	<u>119230.1</u>
13. Hours Reactor Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>709.1</u>	<u>6072.1</u>	<u>114861.0</u>
15. Hours Unit Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>1332240.0</u>	<u>11610480.0</u>	<u>202755216.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>454430.0</u>	<u>3973740.0</u>	<u>68619674.0</u>
18. Net Electrical Energy Generated(MWH)	<u>437127.0</u>	<u>3824083.0</u>	<u>65957777.0</u>
19. Unit Service Factor	<u>95.3</u>	<u>69.3</u>	<u>59.4</u>
20. Unit Availability Factor	<u>95.3</u>	<u>69.3</u>	<u>59.4</u>
21. Unit Capacity Factor (Using MDC Net)	<u>87.7</u>	<u>65.2</u>	<u>50.9</u>
22. Unit Capacity Factor (Using DER Net)	<u>89.7</u>	<u>66.6</u>	<u>52.1</u>
23. Unit Forced Outage Rate	<u>4.7</u>	<u>22.3</u>	<u>12.4</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each) RFO-10, March 25, 1995 for 55 days			
25. If Shutdown at End of Report Period, Estimated Date of Startup - Unit Operating			

BOSTON EDISON COMPANY  
PILGRIM NUCLEAR POWER STATION  
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR DECEMBER 1994

The unit started the reporting period on line at approximately 12 percent Core Thermal Power (CTP) performing activities per the Power Ascension Program schedule. On December 1, 1994 at 0010 hours the turbine generator was taken off-line for alignment and balancing. On December 1, 1994 at 1042 hours the generator was synchronized to the grid. On December 2, 1994 at 1333 hours the turbine generator was again taken off-line for alignment and balancing. On December 3, 1994 at 1355 hours the generator was synchronized to the grid. The Power Ascension program continued and on December 4, 1994 the unit achieved 100 percent CTP where it was essentially maintained until December 6, 1994. On December 6, 1994, power was reduced to approximately 48 percent CTP to swap Augmented Off-Gas (AOG) trains. Following the swap-over, reactor power was increased to 100 percent CTP on December 7, 1994 at 0240 hours. On December 9, 1994 at approximately 2100 hours power was briefly reduced in response to a "A" reactor feedwater pump low seal flow alarm. The unit maintained 100 percent CTP until December 17, 1994 when power was reduced to approximately 50 percent CTP to perform a thermal backwash of the main condenser and performed a control rod pattern change. Following the backwash reactor power was increased, achieving 100 percent CTP at approximately 1100 hours on December 19, 1994 where it was essentially maintained for the remainder of the reporting period.

SAFETY RELIEF VALVE CHALLENGES

MONTH OF DECEMBER 1994

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

## REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to SECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: March 25, 1995.
3. Scheduled date for restart following next refueling: May 19, 1995.
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1993 refueling outage was of the same design as loaded in the previous refueling outage and consisted of 140 assemblies.
7.
  - (a) There are 580 fuel assemblies in the core.
  - (b) There are 1629 fuel assemblies in the spent fuel pool.
8.
  - (a) The station is presently licensed to store 3859 spent fuel assemblies. The actual usable spent fuel storage capacity is 2620 fuel assemblies.
  - (b) The planned spent fuel storage capacity is 3859 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 691 fuel assemblies.

MONTH DECEMBER 1994

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Salt Service Water (SSW) System	SSW Pump P-208D	Low Discharge Head and Pump Vibration in Alert	Normal wear	Pump P-208D overhauled and inspected. Installed rebuilt suction bowl assembly; seven new spider bearings; three line shafts; one head shaft; one stuffing box bearing; packing and new gaskets. FRN 94-03-146 was implemented to hardface the line shafts. Pump was balanced and tested successfully.	To be determined	N/A
Salt Service Water (SSW) System	SSW Loop "B" Auto Start Pressure Switch PS-3829B	Switch malfunction causing false automatic start of SSW Pump P-208C.	Faulty microswitch in Pressure Switch P-3829B	Microswitch replaced in Pressure Switch P-3829B per FRN 94-04-102. Tested satisfactorily.	To be determined.	N/A

## UNIT SHUTDOWNS AND POWER REDUCTIONS DOCKET NO: 50-293

DOCKET NO: 50-293

NAME: Pilgrim I

DATE: 1/13/95

COMPLETED BY: W. Munro

TELEPHONE: (508) 830-8474

REPORT MONTH: December 1994

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONEN CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
10	12/3/94	F	34.9	B	N/A	N/A	N/A	N/A	Power reduction - main generator manually tripped to perform balancing.
11	12/18/94	S	0.0	B	N/A	N/A	N/A	N/A	Power reduction to perform thermal backwash of the main condenser.

1

2

3

4&amp;5

F-Forced  
S-Sched

A-Equip Failure  
 B-Main or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training  
 & License Examination  
 F-Admin  
 G-Operator Error  
 H-Other

1-Manual  
 2-Manual Scram  
 3-Auto Scram  
 4-Continued  
 5-Reduced Load  
 9-Other

Exhibit F & H  
 Instructions for  
 Preparations of  
 Data Entry Sheet  
 Licensee Event Report  
 (LER) File (NUREG-1022)