AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE February 4, 1983
COMPLETED BY T. P. Matthews
TELEPHONE (402)536-4733

MONTH January, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0.0	17	0.0
2	0.0	18	0.0
3	0.0	19	0.0
4	0.0	20	0.0
5	0.0	21	0.0
6	0.0	22	0.0
7	0.0	23	0.0
8	0.0	24	0.0
9	0.0	25	0.0
10	0.0	26	0.0
11	0.0	27	0.0
12	0.0	28	0.0
13	0.0	29	0.0
14	0.0	30	0.0
15	0.0	31	0.0
16	0.0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-285
DATE February 4, 1983
COMPLETED BY T. P. Matthews
TELEPHONE (402)536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: January, 1983
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	744.0	81,985.0
12. Number Of Hours Reactor Was Critical	0.0	0.0	64,110.5
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	0.0	0.0	62,947.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	77,616,548.4
17. Gross Electrical Energy Generated (MWH)	0.0	0.0	25,735,333.5
18. Net Electrical Energy Generated (MWH)	0.0	0.0	24,330,034.4
19. Unit Service Factor	0.0	0.0	76.8
20. Unit Availability Factor	0.0	0.0	76.8
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	64.4
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	64.1
23. Unit Forced Outage Rate	0.0	0.0	3.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: March 20, 1983
26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1983

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE February 4, 1983
COMPLETED BY I. P. Matthews
TELEPHONE (402)536-4733

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-06	821206	S	1350	C	4	N/A	XX	XXXXXX	1982/1983 refueling outage commenced December 6, 1982.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending January 1983 .

1. Scheduled date for next refueling shutdown. — March 1984

2. Scheduled date for restart following refueling. — May 1984

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be?

      A Technical Specification Change

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

   c. If no such review has taken place, when is it scheduled?

4. Scheduled date(s) for submitting proposed licensing action and support information. — Methodology - Dec. 1983; Tech. Specs. - Feb. 1984

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

6. The number of fuel assemblies:

a) in the core	133	assemblies
b) in the spent fuel pool	265	"
c) spent fuel pool storage capacity	483	"
d) planned spent fuel pool storage capacity	728	"

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1985

Prepared by J K Gasper — Date February 1, 1983

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

January, 1983
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station continued refueling outage operations during the month of January.

Several important jobs were completed during January. They include the following:

1. Integrated Leak Rate Test (ILRT). The test was completed successfully and leakage has been calculated to be acceptable.
2. The cycle 7 core was completely offloaded into the spent fuel pool in preparation for the reactor vessel internals inspection.
3. The reactor vessel internals including the core support barrel were removed and stored in the lower cavity area of containment.
4. The ten-year reactor vessel internals inspection was initiated. The start of inspection efforts was delayed slightly due to alignment pin location problems. The alignment pins were relocated to the correct position and inspection efforts are currently in progress.

Several N.R.C. audits were conducted during January in the areas of security, outage health physics, fire protection, and requalification training. American Nuclear Insurers conducted a fire protection inspection and initiated an ongoing inspection covering the reactor vessel internals ten-year inservice inspection.

Mr. A. Richard successfully completed the simulator exam portion of his Senior Reactor Operators license. This completes Mr. Richard's senior license program.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

LER Number	Deficiency
82-021	During performance of "refueling" surveillance test ST-ESF-3, F.2, "Containment Pressure Channel Check", pressure switch A/PC-742-1 was found to initiate above the Technical Specification limit of 5 psig. Pressure switch A/PC-742-1 is one of four switches in the "A" channel of the containment pressure high signal (CPHS) initiation matrix. During the time period A/PC-742-1 would have failed to initiate at or below 5 psig, the other three pressure switches of the "A" channel

A. PERFORMANCE CHARACTERISTICS (Continued)

LER Number	Deficiency
	were operational and a CPHS could have been initiated by two of these three switches actuating. Additionally, the redundant "B" channel pressure switches which initiate the "B" CPHS were fully operable and would have actuated to mitigate the consequences of an accident, if required.

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which resulted in Operation Incidents and are not reported elsewhere in the report:

Operation Incidents		Deficiency
OI-1623	ST-CONT-2, F.1	Missing Documentation for 10/8/81, 10/30/81, 11/9/81, 11/11/81, 11/20/81, 11/21/81, and 11/22/81 as required.
OI-1621	ST-ISI-CVCS-3, F.1	Missed Q.C. verification of calibration of test equipment.
OI-1622	ST-ISI-RW-3, F.1	Test not performed as required during November, 1981. Plant was in refueling outage. Raw Water System could not be aligned to the configuration specified in the test due to maintenance and modifications being performed.
OI-1632	ST-IR-1, F.3	Test results had less than 85% radio-iodine retention.
OI-1636	ST-RM-1, F.3	During the performance of ST-RM-1, F.3, monitor RM-070, 071, 072, 073, 081, 084, 085, and 086 were found to be out of calibration tolerance specification.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Procedure	Description
SP-FH-2	Fuel Hoist Removal from Mast. Not an unreviewed safety question because: The procedure only provides for the orderly removal, repair, and reassembly of the refueling machine hoist box. The appropriate surveillance testing will be performed prior to use.
SP-ECT-1	Eddy Current Testing of Heat Exchanger Tubes. Completed per procedure. This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved eddy current testing of feedwater heaters.

E. RESULTS OF LEAK RATE TESTS

All leak rate tests performed during January and February will be reported to the PRC per a special report (which concerns itself with all CONT-2, 3, 7, and associated tests). This special report will be submitted to the PRC prior to startup.

F. CHANGES IN PLANT OPERATING STAFF

The following promotions were effective January 1, 1983:

Gary L. Roach	Supervisor - Chemical & Radiation Protection
Gerald Chatfield	Shift Supervisor
Joseph M. Mattice	Plant Health Physicist

Two new operators, Dan Hochstein and Dave Harrison, were added to the Operations Department.

G. TRAINING

Training for January, 1983, General Employee Training was conducted several times a week to accommodate the increase in support personnel for the 1983 outage. Operations on-the-job training was conducted on the refueling machine and outage operations. NRC conducted an audit on operator initial and requalification and fire brigade training.

H. CHANGES, TEST AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

Amendment No. 66 adds a temporary authorization to perform secondary tests at a lower temperature.

II. MAINTENANCE (Significant Safety Related)

Refueling outage maintenance will be submitted as one package in April, 1983.

W. G. Gates
Manager
Fort Calhoun Station

Omaha Public Power District

1623 HARNEY ■ OMAHA, NEBRASKA 68102 ■ TELEPHONE 536-4000 AREA CODE 402

February 10, 1983
LIC-83-041

Mr. Richard C. DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Reference: Docket No. 50-285

Dear Mr. DeYoung:

Please find attached ten (10) copies of the January Monthly Operating Report for the Fort Calhoun Station Unit No. 1.

Sincerely,

W. C. Jones
Division Manager
Production Operations

WCJ/TLP:jmm

Attachments

cc: NRC Regional Office
Office of Management & Program Analysis (2)
Mr. R. R. Mills - Combustion Engineering
Mr. T. F. Polk - Westinghouse
Mr. L. A. Yandell - NRC Senior Resident Inspector
Nuclear Safety Analysis Center
NRC File