

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	0	H	D	B	S	1	2	0	0	-	0	0	0	0	-	0	0	3	4	1	1	1	1	4	5						
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32						
LICENSEE CODE														LICENSE NUMBER										LICENSE TYPE						CAT	

CON'T

01	L	6	0	5	0	0	0	3	4	6	7	0	1	1	5	8	3	8	0	2	1	4	8	3	9
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32
REPORT SOURCE		DOCKET NUMBER										EVENT DATE						REPORT DATE							

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 (NP-33-83-03) On 1/15/83 and again on 1/18/83 after reactor trips, the Reactor Coolant System (RCS) Dose Equivalent I-131 exceeded the Technical Specification 3.4.8 limit of 1.0 uCi/gm. The level peaked at 2.67 uCi/gm at 2004 hours on 1/15/83 and at 1.39 uCi/gm at 2301 hours on 1/18/83. There was no danger to the health and safety of the public or station personnel. At no time did the specific activity exceed the allowable limit of Tech Spec 3.4.8 which accommodates possible iodine spiking phenomenon that may occur following a change in thermal power, as shown in Figure 3.4-1

09	R	C	11	E	12	C	13	F	U	E	L	E	X	14	Z	15	Z	16	17	8	3	21	22	0	0	2	23	0	3	28	29	L	30	0	32					
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47
LER/RO REPORT NUMBER		EVENT YEAR		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE						COMP. SUBCODE		VALVE SUBCODE		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.																
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS						ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER																				
Z		Z		Z		Z		0000						Y		N		N		B015																				

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause was a slight leakage of fission products through the fuel cladding. Some leakage is normal following a reactor trip due to RCS temperature and pressure changes. The I-131 level was monitored until it dropped below the Technical Specification limit.

15	X	28	0	0	0	29	NA	30	A	31	Per Table 4.4-4 of Tech Spec 3.4.8	32
7	8	9	10	11	12	13	14	15	16	17	18	19
FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION				

16	Z	33	Z	34	NA	35	NA	36	NA	37
7	8	9	10	11	12	13	14	15	16	17
ACTIVITY RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE						

17	0	0	0	37	Z	38	NA	39
7	8	9	10	11	12	13	14	15
PERSONNEL EXPOSURES		TYPE		DESCRIPTION				

18	0	0	0	40	NA	41
7	8	9	10	11	12	13
PERSONNEL INJURIES		TYPE		DESCRIPTION		

19	Z	42	NA	43	
7	8	9	10	11	
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION	

20	N	44	NA	45	
7	8	9	10	11	
ISSUED		DESCRIPTION		PDR	

TOLEDO EDISON COMPANY
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE
SUPPLEMENTAL INFORMATION FOR LER NP-33-83-03

DATE OF EVENT: January 15 and 18, 1983

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Reactor Coolant System (RCS) Dose Equivalent I-131 Exceeded Technical Specification Limits

Conditions Prior to Occurrence: The unit was in Mode 3, with Power (MWT) = 0 and Load (Gross MWE) = 0

Description of Occurrence: On January 15, 1983 after a reactor trip, the specific activity of the primary coolant I-131 exceeded the Technical Specification 3.4.8 limit of 1.0 $\mu\text{Ci/gm}$ Dose Equivalent I-131. The leak peaked at 2.67 $\mu\text{Ci/gm}$ at 2004 hours on January 15, 1983.

A similar event occurred after a reactor trip on January 18, 1983 with the dose equivalent level peaking at 1.39 $\mu\text{Ci/gm}$ at 2301 hours.

The following information is supplied per reporting requirements:

Power History

On January 15, 1983, reactor power was approximately 100% until a reactor/turbine trip at 1700 hours.

<u>Date</u>	<u>Time</u>	<u>Reactor Power</u>
1/14/83	0000 hours	99.4%
	1600 hours	99.5%
1/15/83	1600 hours	98.7%
	1700 hours	TRIP

On January 18, 1983, reactor power was approximately 100% until a reactor/turbine trip at 1925 hours.

<u>Date</u>	<u>Time</u>	<u>Reactor Power</u>
1/16/83	1600 hours	84.3%
1/17/83	0000 hours	87.3%
	0800 hours	97.9%
	1600 hours	99.2%
1/18/83	0000 hours	99.3%
	0800 hours	99.4%
	1600 hours	99.6%
	1925 hours	TRIP

Fuel Burnup By Core Region: See attached computer printout.

Cleanup Flow History 48 hours prior to the first limiting exceeding sample, the average letdown flow during this time was:

January 13, 1983 - 60 gpm
January 14, 1983 - 61 gpm
January 15, 1983 - 86 gpm

January 16, 1983 - 109 gpm
January 17, 1983 - 101 gpm
January 18, 1983 - 60 gpm

There was no degassing operation.

Table 1 lists the specific activity analysis, the time duration the specific activity exceeded the Technical Specification, and the maximum level reached for each occurrence.

Designation of Apparent Cause of Occurrence: The cause was a slight leakage of fission products through the fuel cladding. Some leakage is normal following a reactor trip due to RCS temperature and pressure changes.

Analysis of Occurrence: There was no danger to the health and safety of the public or station personnel.

At no time did the specific activity exceed the allowable limit of Technical Specification 3.4.8, which accommodates possible iodine spiking phenomenon that may occur following a change in thermal power as shown in Figure 3.4-1.

Corrective Action: Per the action statement requirements of Technical Specification 3.4.8, sampling of the primary coolant was performed at least once every four hours. Monitoring of the I-131 level continued until it dropped below the Technical Specification limit of 1.0 $\mu\text{Ci/gm}$. This occurred at 1730 hours on January 16, 1983 when the level dropped to 0.73 $\mu\text{Ci/gm}$ for the first occurrence. At 0645 hours on January 19, 1983, the level dropped to 0.81 $\mu\text{Ci/gm}$. Since iodine spikes are typical following a power change, no additional corrective action is applicable.

Failure Data: Previous occurrences of the RCS dose equivalent I-131 exceeding the Technical Specification limit have been reported in Licensee Event Reports NP-33-80-114 (80-088), NP-33-81-15 (81-016), NP-33-81-37 (81-031).

LER #83-002

TABLE 1

<u>Date of Occurrence</u>	<u>Specific Activity Analysis</u>			<u>Approximate Time Duration Tech Spec Level Exceeded</u>	<u>Maximum Level $\mu\text{Ci/gm}$</u>
	<u>Date</u>	<u>Time</u>	<u>Activity ($\mu\text{Ci/gm}$) Dose Equivalent I-131</u>		
January 15, 1983	1/15/83	0810	0.17	21 hours	2.67
		2004	2.67		
		2115	2.54		
	1/16/83	0102	2.33		
		0400	2.40		
		0730	1.59		
		0930	1.58		
		1330	1.14		
		1730	0.73		
January 18, 1983	1/18/83	1354	0.14	8 hours	1.39
		2301	1.39		
	1/19/83	0256	1.11		
		0645	0.81		

FUEL BURNUP BY CORE REGION FOR
JANUARY 15, 1983 AND JANUARY 18, 1983 EVENTS

The fuel burnup is given for each fuel assembly as designated by the computer assigned assembly number.
For the location of the assemblies, see the attached core map. Units are given in $\frac{\text{Megawatt Days}}{\text{Metric Ton}} \times 10^{-3}$.

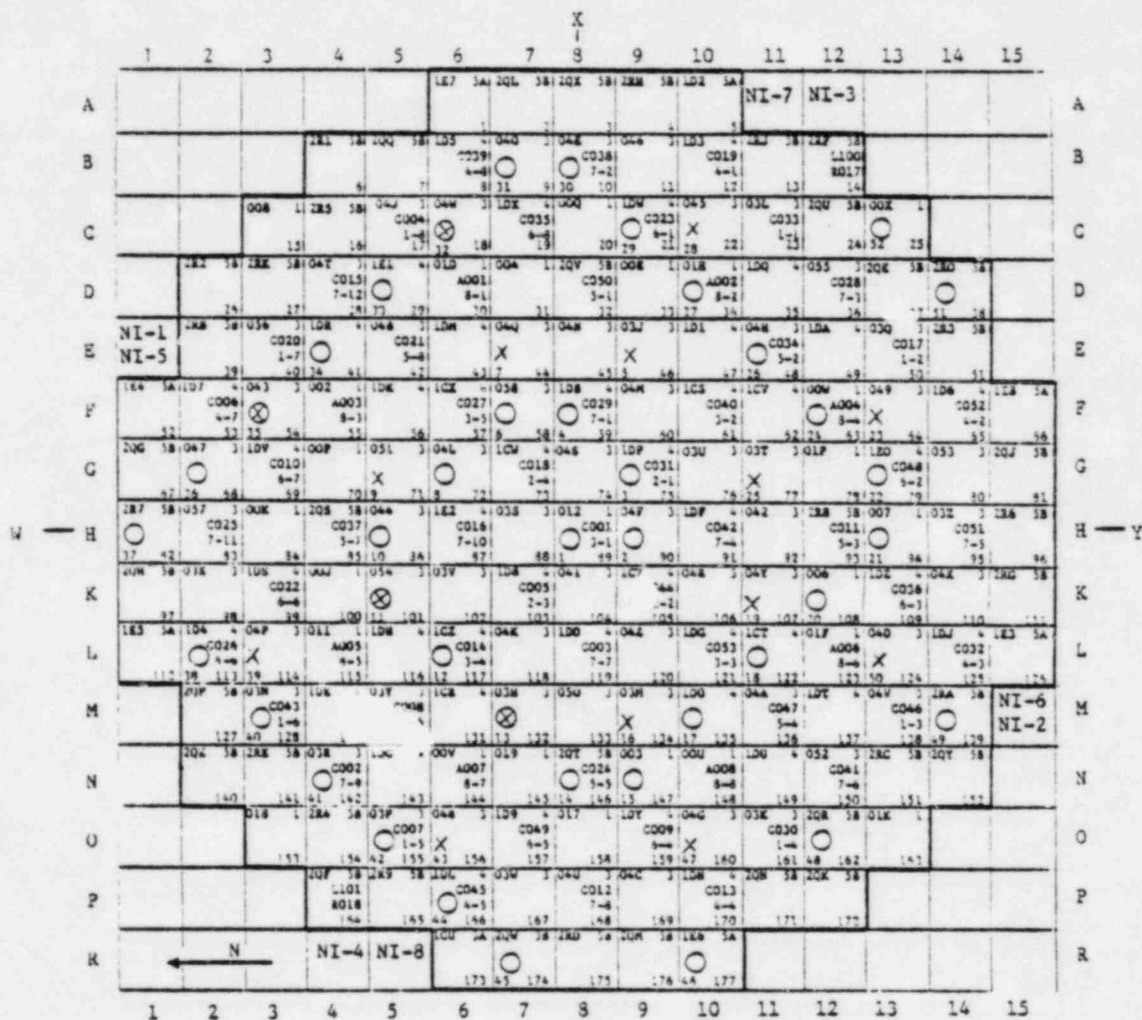
								FUEL ASSEMBLIES*
27542E+01	35514E+01	37802E+01	35488E+01	27457E+01	29172E+01	39053E+01		1 through 7
11194E+02	22218E+02	17817E+02	22230E+02	11193E+02	38853E+01	28741E+01		8 through 14
13450E+02	45135E+01	24701E+02	24023E+02	15512E+02	15337E+02	15520E+02		15 through 21
23979E+02	25082E+02	44478E+01	13404E+02	25487E+01	45411E+01	26106E+02		22 through 28
14377E+02	15475E+02	16534E+02	51114E+01	16536E+02	15474E+02	14349E+02		29 through 35
26059E+02	44299E+01	28539E+01	35650E+01	25144E+02	14499E+02	24408E+02		36 through 42
12306E+02	27081E+02	28112E+02	27058E+02	12296E+02	24388E+02	14431E+02		43 through 49
24681E+02	38423E+01	27937E+01	11046E+02	23667E+02	15591E+02	12185E+02		50 through 56
13853E+02	26236E+02	13941E+02	26236E+02	13844E+02	12157E+02	15551E+02		57 through 63
23477E+02	10931E+02	27054E+01	35837E+01	22635E+02	15195E+02	16319E+02		64 through 70
27439E+02	26967E+02	13561E+02	22297E+02	13566E+02	26989E+02	27494E+02		71 through 77
16303E+02	15143E+02	22564E+02	34890E+01	37867E+01	17817E+02	15302E+02		78 through 84
59180E+01	28009E+02	13888E+02	22272E+02	15965E+02	22294E+02	13917E+02		85 through 91
28070E+02	50464E+01	15290E+02	17769E+02	37149E+01	35332E+01	22574E+02		92 through 98
15124E+02	16265E+02	27458E+02	26941E+02	13520E+02	22247E+02	13530E+02		99 through 105
26987E+02	27506E+02	16310E+02	15137E+02	22563E+02	34842E+01	27244E+01		106 through 112
10929E+02	23423E+02	15508E+02	12686E+02	13768E+02	26177E+02	13878E+02		113 through 119
26153E+02	13790E+02	12141E+02	15548E+02	23476E+02	10917E+02	26961E+01		120 through 126
38357E+01	25225E+02	14377E+02	24304E+02	12232E+02	27029E+02	28035E+02		127 through 133
26958E+02	12198E+02	24336E+02	14401E+02	25609E+02	38225E+01	28543E+01		134 through 140
44076E+01	26031E+02	14309E+02	15442E+02	16491E+02	50199E+01	16460E+02		141 through 147
15423E+02	14297E+02	26028E+02	44067E+01	28479E+01	13399E+02	44260E+01		148 through 154
24797E+02	23939E+02	15484E+02	15300E+02	15461E+02	23929E+02	25181E+02		155 through 161
44202E+01	13398E+02	28705E+01	38691E+01	11172E+02	22207E+02	17793E+02		162 through 168
22196E+02	11163E+02	38608E+01	28678E+01	27357E+01	35349E+01	37673E+01		169 through 175
35331E+01	27324E+01							176 through 177

*AS READ FROM LEFT TO RIGHT

TOLEDO EDISON COMPANY
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE
SUPPLEMENTAL INFORMATION OFR LER NP-33-83-03
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DAVIS-BESSE UNIT 1, CYCLE 3



Incore Detector
○ = Total Core Monitor
X = Symmetry Monitor
⊗ = Total Core and Symmetry Monitor

Incore Detector String Number

Regenerative Sources at P4 and B12

Computer Assigned Assembly Number

Fuel Assembly ID
All ID's preceded by "NJ0"

Fuel Assembly Batch ID, (No. in Core)
1 (25) Former Batch A - once burned
3 (60) Former Batch C - twice burned
4 (44) Former Batch D - once burned
5A (8) New Fuel - 3.04% enriched
5B (40) New Fuel - 2.99% enriched

Control Component/Source ID and Type
A = APSRA C = CRA
R - Regenerative Neutron Source

Control Component Group/Rod

Prepared By

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Approved By

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