

TEXAS UTILITIES SERVICES INC.

2001 BRYAN TOWER DALLAS, TEXAS 75201-3050

Log # TXX-3621
File # 10010

February 14, 1983

Mr. S. B. Burwell
Licensing Project Manager
U. S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Washington, D.C. 20555

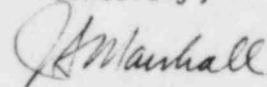
SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION
DOCKET NOS. 50-445 & 50-446
FSAR AMENDMENT 38 DESCRIPTION

Dear Mr. Burwell:

Please find attached a table providing the description of Amendment 38,
dated February 14, 1983.

If you have any questions or comments, please call.

Sincerely,



J. S. Marshall

RWH:tls
Attachment

cc: H. C. Schmidt

BOOI

CPS&S/FSAR

AMENDMENT 38 DESCRIPTION

February 14, 1983

<u>SECTION</u>	<u>DESCRIPTION OF CHANGE</u>
1.2	Removes references to Seismic Category I from fire protection piping (Changed to Class 5).
T1.7-1	Provides update to RCS high point vent system.
1A(B)	Upgrades latest edition of ANSI/ASME N-509 instead of earlier versions referenced in R.G. 1.140.
3.5	Incorporates a new reference (Bechtel topical report).
3.7B	Provides clarification on when the simplified design method is used.
3.8	Deletes reference of ASTM Spec. A20-74b, "Specification for General Requirements for Steel Plates for Pressure Vessels". Steel plates for pressure vessels are not elements of containment internal structures.
3.9B	Refer to 3.7B.

<u>SECTION</u>	<u>DESCRIPTION OF CHANGE</u>
3.11N	Provides update to explicitly define compliance with GDC-4 for mechanical equipment qualification.
3.11B	Provides update to explicitly define compliance with GDC-4 for mechanical equipment.
	Provides update to postulated environments.
T3.11B-2	Provides update to postulated environments.
APP 3A T4-1 & T5-1	Provides update to Environmental Qualification tables.
5.1	Provides update to RCS high point vent description.
6.2	Provides clarification of hydrogen calculations.
	Provides change to description of containment boundary and testing requirements as they pertain to the valve isolation tanks.

<u>SECTION</u>	<u>DESCRIPTION OF CHANGE</u>
	Deletes specific weight information for galvanized material. Original figures were indicative of nominal weight.
T6.2.4-3	Provides correction of "Valve Closure Time". These valves do not receive containment isolation signals.
T6.2.5-3	Deleted as a result of Hydrogen Reanalysis (A-36).
6.3	Provides correction to the description of safety injection pump injection lines to be tested.
7.3	Provides correction to description for the control of AFW System valves.
T7.5-7	Provides update to the range of RHR flow meter.
7.7	Provides update to reflect actual turbine runback valves.
8.3	Provides updates to the capacities for batteries BTIED3 and 4.
9.1	Provides editorial changes.

<u>SECTION</u>	<u>DESCRIPTION OF CHANGE</u>
9.2	Provides clarification of SSWs w.r.t. sharing. (i.e. traveling screens). Clarification of SSWs w.r.t. heavy loads.
9.5	Removes "electric fire pump failure to start" as a cause of actuation of diesel fire pump. Provides update to fire loads (Fire Zone 19) Provides title changes in operations.
11A	Provides update on current water use.
13.1	Provides startup organizational changes.
F13.1-2	"
F13.1-5	"
13.2	Provides update of general employee training programs.
13.5	Provides update on plant operating procedures.
T13.5-1	"

<u>SECTION</u>	<u>DESCRIPTION OF CHANGE</u>
T13.5-2	"
T13.5-3	Provides update on plant operating procedures.
T13.5-4	"
T13.5-5	"
T13.5-9	"
14.2	Provides startup organizational changes.
T14.2-2	Provides update to test procedures for fire protection, startup transformers, nuclear instrumentation and AFW System.
F14.2-1	Provides startup organizational changes.
F17.1-6	Provides update to organizational chart.
F17.2-1	Provides update to TUGCo organizational chart.
17A	Provides update to Q-list.

<u>SECTION</u>	<u>DESCRIPTION OF CHANGE</u>
T17A-1	Provides update to RCS high point vent system.
	Provides clarification of component quality requirements.
II.B.1	Provides update to RCS high point vent system description.
II.F.2	Provides update to inadequate core cooling monitoring instrumentation system.
TII.F.2-1	"
II.K.2.13	Provides update of CPSES response to "Effect of High Pressure Injection on Vessel Integrity for Small Break Loss-of-Coolant Accident With No Auxiliary Feedwater".
II.K.2.17	Provides update of CPSES response to "Analyze the Potential for Voiding in Reactor Coolant System During Anticipated Transients".
II.K.3.30	Provides update of CPSES response to "Small Break Loss of Coolant Accident Methods to Show Compliance with 10 CFR Part 50 Appendix K".
032	Provides updated response to 032.106 on loss of instrumentation and control buses.