

## SNUPPS

Standardized Nuclear Unit  
Power Plant System

5 Choke Cherry Road  
Rockville, Maryland 20850  
(301) 869-8010

January 18, 1984

SLNRC 84-0005 FILE: 0541/M-189  
SUBJ: NRC Request for Information re  
Preservice Inspection (PSI)  
Relief Request: Callaway Plant

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Docket Nos. STN 50-482 and STN 50-483

Dear Mr. Denton:

The purpose of this letter is to advise NRC of the categories of preservice inspection (PSI) examination at Callaway Plant for which relief requests are anticipated. This information is provided in response to NRC telecon request of January 9, 1984. Specific PSI examination categories requiring relief are as follows:

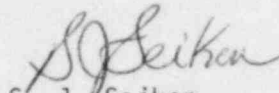
1. Reactor Coolant System Small Bore Piping: A total of 16 small bore pipe welds in the Reactor Coolant System could not be meaningfully examined and will require relief from existing UT examination plans and commitments. All welds in this category involve Class II piping with pipe diameter of 1.5 inches or less. A listing of specific welds is provided in Attachment A to this letter. A supplemental PT examination was performed on each weld listed in the attachment.
2. Random Component and Piping Welds: The second category of relief request involves limited or partial PSI examination of selected component and piping welds. These welds are randomly located throughout the plant. The Callaway inspection agency, NES, plans to conduct a complete review of Incomplete Examination Reports generated at Callaway and will advise SNUPPS/Union Electric of the total number of welds requiring a partial examination exemption and the individual weld identification numbers of each. This information will be forwarded to NRC in early February.
3. Reactor Pressure Vessel Examination: While the examination of the Callaway Reactor Pressure Vessel has yet to be completed, the experience of the inspection agency is that examination coverage of the vessel may be limited in the areas of the core support lugs. Examination in this area will start later this month and is scheduled to be completed by February 28th. NRC will be advised within 30 days of completion of the vessel examination of those welds, if any, requiring a partial relief request.

Boo1

1/1

We trust the preceding information is responsive to NRC needs. If further information or clarification of the information provided herein is required, please contact the undersigned. In the event there is need for additional relief requests involving Callaway PSI examination, the NRC will be so advised. Information concerning relief requests for Wolf Creek Plant will be provided at a later date.

Very truly yours,



S. J. Seiken  
Manager, Quality Assurance

SJS/dck/24a2

Attachment: Listing of RCS Small Bore Pipe Welds

cc: J. Neisler/B. Little, USNRC/CAL  
W. Schum/K. Whittlesey, USNRC/WC  
J. Konklin, USNRC/RIII  
D. T. McPhee, KCPL w/o  
G. L. Koester, KGE w/o  
D. F. Schnell, UE w/o

Listing of RCS Small Bore Pipe Welds

| <u>Weld Number</u> | <u>System</u>   | <u>Supplemental<br/>Examination</u> |
|--------------------|-----------------|-------------------------------------|
| 2-BG09-FW 432      | Reactor Coolant | PT                                  |
| 2-BG09-FW 431      | Reactor Coolant | PT                                  |
| 2-BG09-FW 430      | Reactor Coolant | PT                                  |
| 2-BG09-FW 429      | Reactor Coolant | PT                                  |
| 2-BG09-FW 417      | Reactor Coolant | PT                                  |
| 2-BG09-FW 416      | Reactor Coolant | PT                                  |
| 2-BG09-FW 415      | Reactor Coolant | PT                                  |
| 2-BG09-FW 414      | Reactor Coolant | PT                                  |
| 2-BG09-FW 402      | Reactor Coolant | PT                                  |
| 2-BG09-FW 401      | Reactor Coolant | PT                                  |
| 2-BG09-FW 400      | Reactor Coolant | PT                                  |
| 2-BG09-FW 399      | Reactor Coolant | PT                                  |
| 2-BG09-FW 387      | Reactor Coolant | PT                                  |
| 2-BG09-FW 386      | Reactor Coolant | PT                                  |
| 2-BG09-FW 385      | Reactor Coolant | PT                                  |
| 2-BG09-FW 384      | Reactor Coolant | PT                                  |