

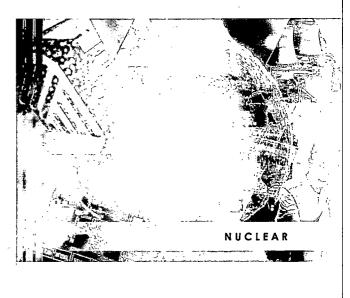
# BWRVIP-316: BWR Internals Aging Management for Extended Operations

(CLOSED SESSION)

Wayne Lunceford, P.E. Technical Executive, EPRI BWRVIP

NRC Public Meeting March 25, 2020

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# Agenda (Closed Session)

BWRVIP-74-A Background and Relationship to BWRVIP-316

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- Scope of Components Evaluated
- Degradation Assessment
- Aging Management Review for Extended Operations
- AMP Attribute Assessment for Extended Operations
- Upper Shelf Energy (USE) Evaluations
  - RPV Plates and Welds

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- RPV Nozzle Forgings
- Status & Plans for Submittal

### **BWRVIP-74-A Background**

- BWRVIP-74-A, RPV I&E Guidelines for License Renewal, provides a demonstration that aging effects applicable to RPV components will be adequately managed for an initial license renewal period
  - Although identified as an "I&E Guideline", the report does not specify inspection requirements
  - Documents that adequate aging management of RPV components is accomplished through an "integrated" approach that relies on Regulation, ASME B&PVC requirements, and supplemental inspections specified within existing BWRVIP I&E guidelines
  - No new inspections were determined to be needed
- Initial version of BWRVIP-74 developed prior to the initial GALL Report (NUREG-1801) and provided a basis that NRC could use to assess LRAs
  - NUREG-1801 references BWRVIP-74 in addressing aging management of RPV neutron embrittlement and recommends aging management of RPV components using an approach consistent with that in BWRVIP-74-A

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### **BWRVIP-316 Key Elements**

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- Revisits the conclusions presented in BWRVIP-74-A regarding RPV aging management
  - Focuses on BWRVIP aging management guidance
  - Assesses guidance to identify time-dependent factors
  - Assesses the applicability of BWRVIP guidance for extended operations
- Provides a high level "roadmap" of BWRVIP guidance relevant to RPV aging management
- Provides generic evaluations that can be used by licensees to address select RPV TLAAs

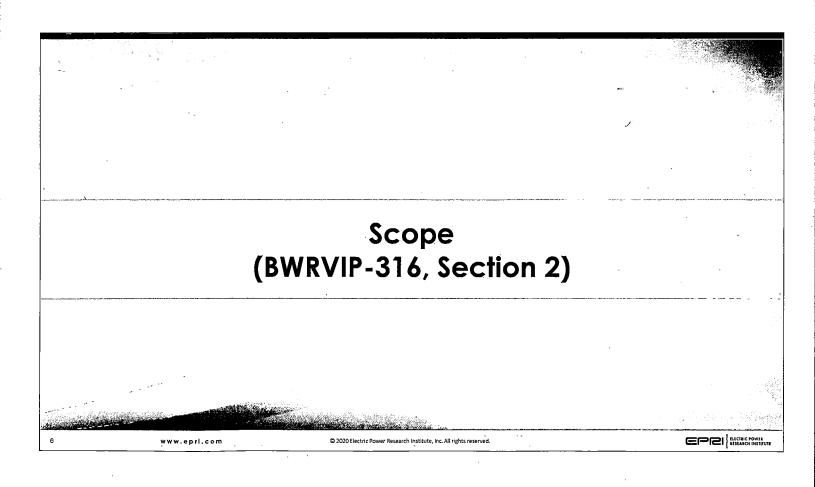
#### Relationship Between BWRVIP-74-A and BWRVIP-316

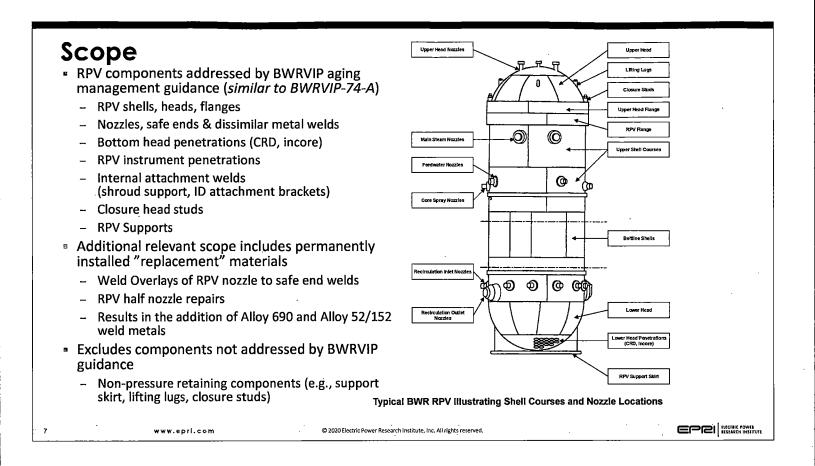
- BWRVIP-316 does not replace BWRVIP-74-A
  - Licensees may still credit the NRC-approved conclusions and apply the NRC-approved evaluation methods contained in BWRVIP-74-A in SLRAs
- Instead, BWRVIP-316:

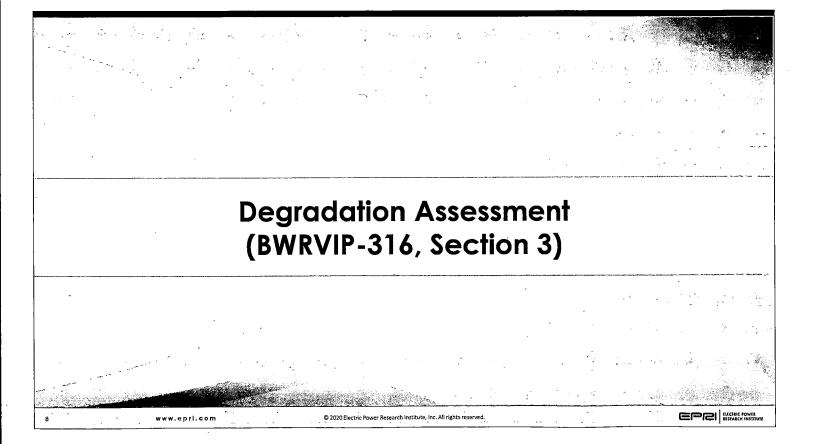
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- Confirms that the integrated set of inspection and flaw evaluation requirements identified in BWRVIP-74-A remain adequate to address operation beyond 60 years
- Recognizes the extension of the integrated surveillance program (ISP) to address 80-year operations

 Summarizes alternative methods that may be applied by licensees to address generic TLAAs







	Structural Materials
arbon and Low-Alloy Steels	A-302, Gr. B, SA-533, Gr. B, Cl. 1, SA-336, SA-508, Cl. 1, SA-508, Cl. 2
Stainless Steel and weld filler materials)	Types 304, 304L, 316, and 316L (weld materials: 308, 308L, 309, 309L, 316, 316L)
Ni-base Alloys	Alloys 600, 690 (Weld materials: Alloys 82, 182, 52, 152)
	Fasteners & Hardware
Low Alloy Steels	ASME SA-193 or SA-540/540M, Grade B23 or B24

## Applicable Aging Effects and Associated Degradation Mechanisms

Aging Effect	Age-Related Degradation Mechanisms		
	General Corrosion		
oss of Material	Crevice Corrosion		
	Wear		
Caralita-	Stress Corrosion Cracking (SCC, IGSCC)		
Cracking	Low-Cycle Fatigue (LCF) / Environmentally-Assisted Fatigue (EAF)		
Loss of Fracture Toughness	Neutron Embrittlement		

#### Extended Operations Aging Management Impact Evaluation

- Identify those aging effects and associated age-related degradation mechanisms that require additional review to assess the impact of operation beyond 60 years (extended operations) on the adequacy of existing BWRVIP aging management guidance
- Where the potential for new occurrences of age-related degradation (e.g., initiation of new cracks) or for progression of degradation (e.g., crack growth rates) does NOT correlate with operating time, there is no reason to revisit prior conclusions regarding the adequacy of existing BWRVIP aging management guidance
- For degradation mechanisms not impacted by consideration of extended operations, continued implementation of existing aging management guidance is adequate, regardless of accumulated service time

#### **Extended Operations Impact Review Results**

Aging Effect	Age-Related Degradation Mechanisms
	General Corrosion
Loss of Material	Crevice Corrosion
	Wear
Craching	Stress Corrosion Cracking (SCC, IGSCC)
Cracking	Low-Cycle Fatigue (LCF) / Environmentally-Assisted Fatigue (EAF)
Loss of Fracture Toughness	Neutron Embrittlement

Potential Impact

No Impact

# Degradation Mechanisms Not Impacted by Extended Operations (1 of 2) Loss of Material General corrosion rates are not significant for RPV components Corrosion allowances far exceed anticipated service lives Excessive crevice corrosion of stainless steels and nickel alloys will not occur irrespective of exposure time provided that water chemistry controls are

- maintainedManaged by water chemistry program
- Wear
  - No basis for postulating time-dependent acceleration or significant adverse trends with extended operating periods
  - Although an adverse trend could potentially occur if a plant undergoes significant changes in operation (such as a power uprate or flexible operations), changes in trends are not truly time-dependent, but are a result of the change in operating conditions

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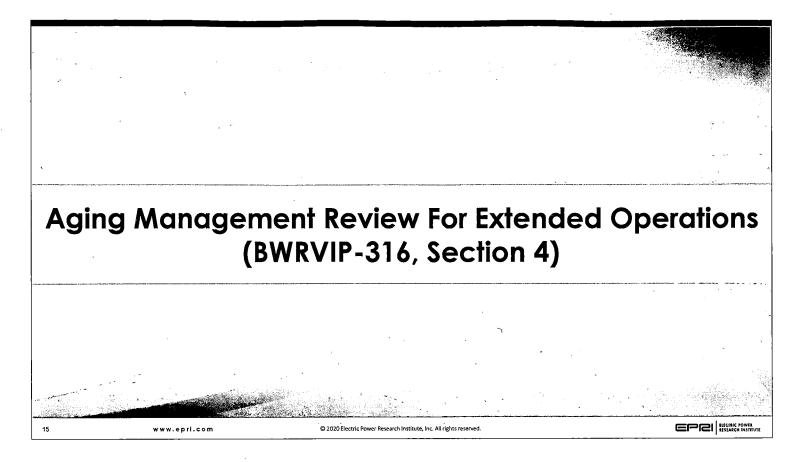
# Degradation Mechanisms Not Impacted by Extended Operations (2 of 2)

SCC / IGSCC

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- For austenitic materials, the results of a review of RPV OE indicates a declining or flat trend
- Two occurrences associated with RPV water level instrumentation
- Limited instances of IGSCC identified in nozzle to safe end welds (many of which are likely not new indications, but newly identified through implementation of improved UT techniques)
- No reason to anticipate any significant change in IGSCC occurrence or growth trends
- CONCLUSION AGING MANAGEMENT GUIDANCE ADDRESSING IGSCC REMAINS ADEQUATE FOR EXTENDED OPERATIONS



# **BWRVIP I&E Guidance**

	Document	RPV Component(s) Addressed	Augmented Exam Reqd?
	BWRVIP-27	Standby Liquid Control (SLC) Nozzles and Safe Ends	Υ [1]
	BWRVIP-38	Shroud Support	Υ [2]
	BWRVIP-47	Bottom Head Penetrations	N <sup>[3]</sup>
	BWRVIP-48	ID Attachment Bracket Welds	Y [4]
:	BWRVIP-49	Penetrations	N <sup>[3]</sup>
[1] Vol m	lumetric examination etal welds and heavil	or enhanced leakage inspection of nozzle to safe end dissimilar y cold worked safe end extensions.	1 . <i>.</i>
[2] UT	/ EVT-1 of shroud su	pport plate to RPV welds and gusset to RPV welds (some designs)	
[3] Exi	sting ASME Section X	l activities are adequate.	
[4] EV ste	T-1 examination of co eam dryer support an	pre spray piping bracket welds, jet pump riser brace bracket welds, d feedwater bracket attachments which use furnace-sensitized stainless steel or Allo	y 182 weld metal.
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#### **Relevant Operating Experience**

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- Operating experience includes very few instances of cracking in these locations, including:
  - Two instances of RPV instrument penetration cracking
  - Several instances of Alloy 600 / 182 stub tube cracking in a BWR/2 (no further instances for many years now)
  - Limited cases of steam dryer support bracket weld cracking (furnace sensitized stainless steel material)
- There is no time dependent trend observed and therefore, no basis to conclude that existing guidance should be modified to address extended operations

### Time Dependent Inputs / Limitations Associated with BWRVIP I&E Guidance

- Comprehensive review of BWRVIP reports and correspondence performed
- Two potentially relevant time-dependent evaluations for the shroud support:
  - BWRVIP 2006-334 includes a fracture mechanics evaluation for the shroud support, including the H9 support plate to RPV weld. The evaluation is not time-dependent because the result is used only to demonstrate that proposed inspection intervals are conservative.
  - BWRVIP-104: PFM analysis assuming a 40-year plant life. The evaluation is not credited as a basis for the inspection intervals specified in BWRVIP-38.
- Inspection requirements are interval-based (independent of plant service time)

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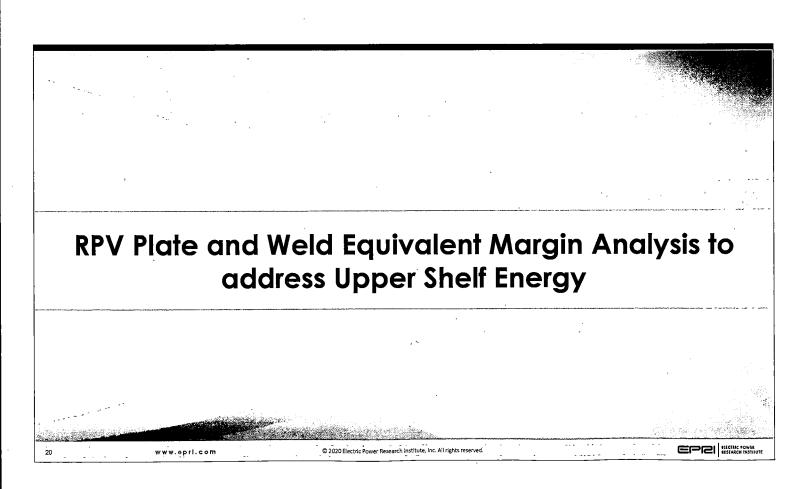
- Therefore, no changes to BWRVIP I&E guidance were determined to be needed to address extended operations
- Minor clarifications may be made, but are not required

## **BWRVIP Evaluations Supporting RPV TLAAs**

- Inservice Inspection of RPV Circumferential Welds
  - Evaluation documented in BWRVIP-329, Updated Probabilistic Fracture Mechanics Analyses for BWR RPV Welds to Address Extended Operations
- USE Evaluation for RPV Plates and Welds
  - A 60-year equivalent margins analysis (EMA) is included in Appendix B of BWRVIP-74-A
  - Review of the EMA in BWRVIP-74-A determined that sufficient conservatism exists to address extended operations
  - Verification forms were restructured to maximize use of the existing EMA results
- USE Evaluation for RPV Nozzle Forgings

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- Reduction in nozzle forging USE is a TLAA for some plants
- In cases, there may be limited data indicative of true upper shelf properties
- Evaluations of available nozzle forging data included in BWRVIP-316 conclude that there is no evidence of a safety concern associated with nozzle forging USE reduction

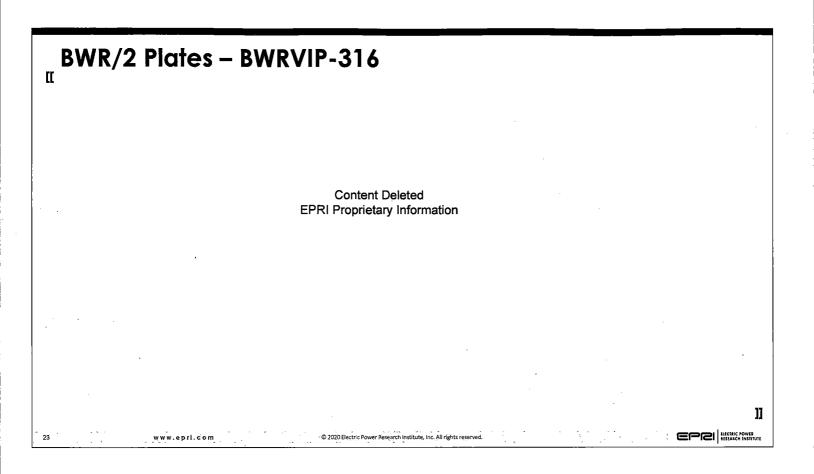


#### **Issue Summary**

- BWRVIP-74-A, Appendix B contains criteria for demonstrating adequate material toughness
- Based on equivalent margin analyses (EMA) developed in the context of 60year operation
- Review of this analysis indicates sufficient conservatism exists in the "60-year" evaluation in BWRVIP-74-A to address extended operations
- BWRVIP-74-A results arbitrarily limited by maximum predicted USE % drop in fleet based on Reg. Guide 1.99
- BWRVIP-316 results reflect maximum allowable % USE drop to reach EMA limit. Either:
  - Minimum value from U.S. fleet database
  - Lower bound value based on 95% confidence
- Verification form removes use of "54 EFPY", now simply states "fluence"

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Results are presented on the following slides



# BWR/3-6 Plates – BWRVIP-74-A

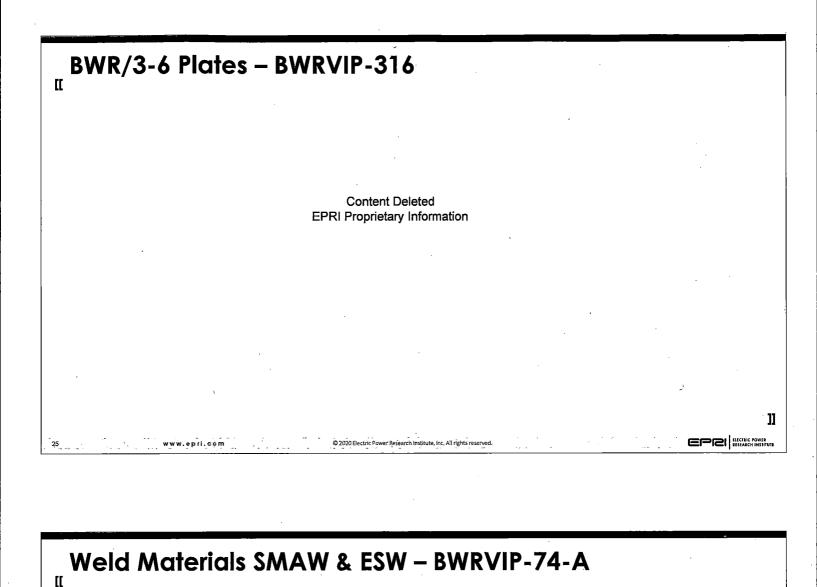
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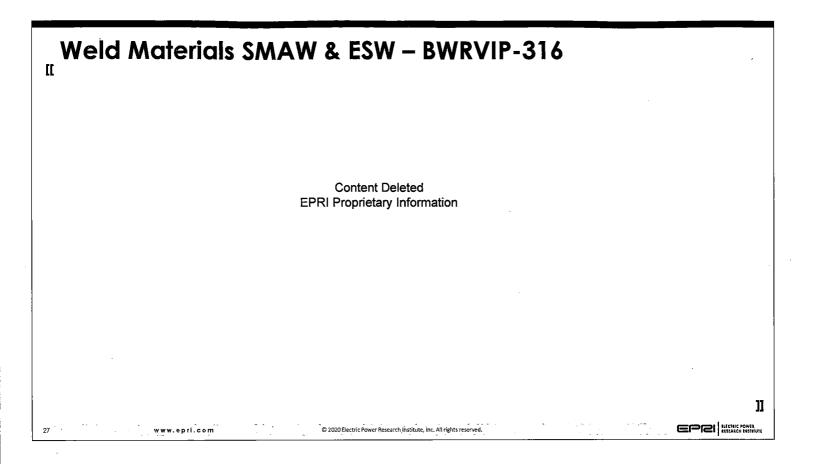
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# $\underset{III}{\mathsf{Weld Materials SAW}} - \mathsf{BWRVIP}{-74}{-}\mathsf{A}$

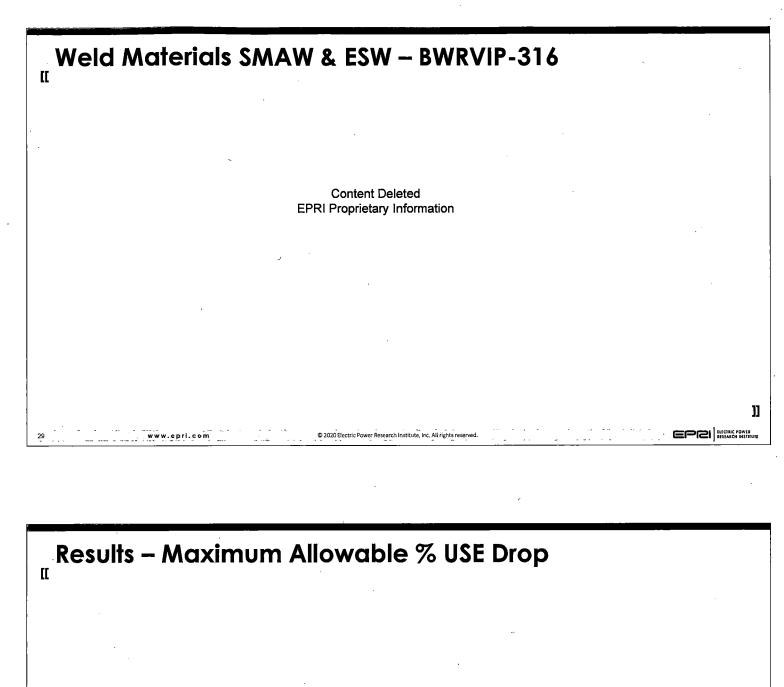
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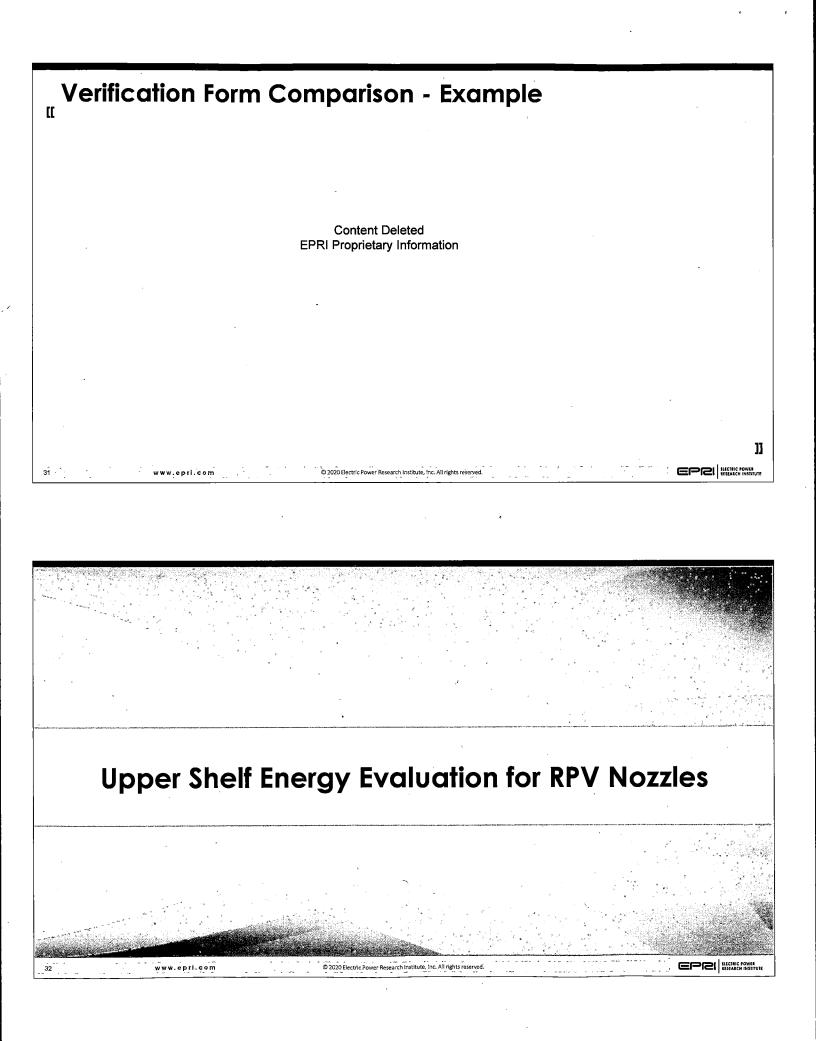


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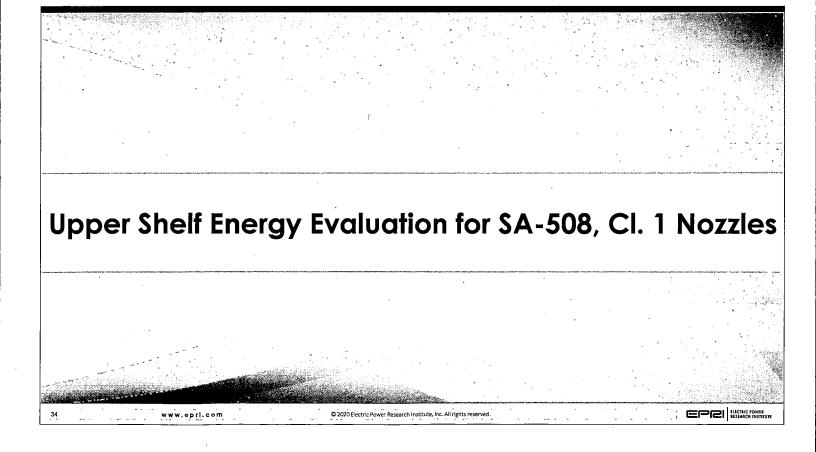
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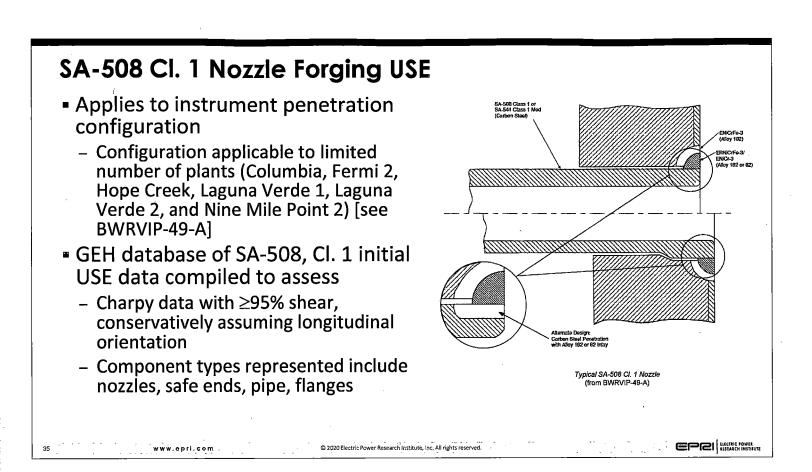


# **RPV Nozzle Upper Shelf Energy (USE)**

- TLAA that has been previously identified for initial LR
- 10CFR50 Appendix G requires EOL USE  $\geq$  50 ft-lbs
- Consideration of SLR results in a need for plants to revisit this issue
- Two nozzle types need to be considered:
  - SA-508 Cl. 1 instrument nozzles
  - SA-508 Cl. 2 nozzles



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#### SA-508 Class 1 Initial USE Data

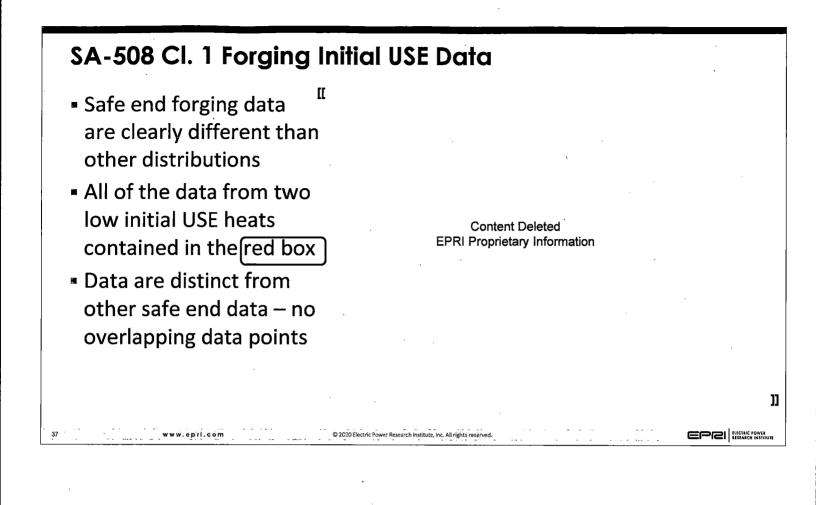
- Atypical distribution of results
- Many high USE data points truncated by machine test capability
- All of the low initial USE test results associated with two heats of material used for safe end forgings
- All of the nozzle and pipe data have high initial USE

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### SA-508 Class 1 Initial USE Data

- Considering all data ([[ ]] data points)
  - Mean = [[
  - Min measurement = [[ ]]
  - Mean- $2\sigma = [[ ]]$
- Considering only data for nozzles and pipe ([[ ]] data points)

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- Mean = [[
- Min measurement = [[
- Mean-  $3\sigma = [[ ]]$
- Considering only data from two low USE heats (safe ends) ([[ ]] data points)
   Mean = [[ ]]
  - Mean = [[ ]]
     Min measurement = [[ ]]
  - Mean-  $2\sigma = [[ ]]$

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 Nonparametric statistical test concludes safe end data are from a different population than nozzle and pipe data

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#### Discussion

- An investigation into the low toughness heats was performed to identify any characteristics that would explain the differences. No such evidence was identified. However, these two heats are clearly different than the remaining 26 heats in the database.
- Resources likely not expended to investigate low initial USE values, so long as the result obtained was acceptable
- Minimum initial USE needed to satisfy a bounding assessment is only 65 ft-lbs
  - Assuming Cu content at upper end of specification allowable (0.35%) and a conservative upper-end fluence associated with 80-yr operations
- The consequence of low upper shelf toughness is not the same for these penetrations as for RPV shells and large diameter nozzles
  - In the very unlikely event that the penetration or extension/safe end cracked 360° and separated, the resulting small break would be immediately detected by drywell pressure, temperature or leakage detection instrumentation and a safe reactor shutdown would be achieved

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- This assessment is included within BWRVIP-49-A

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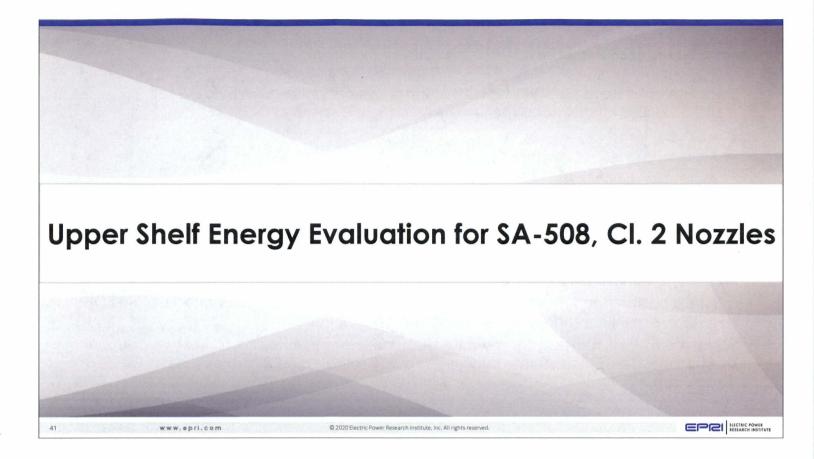
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#### **BWRVIP Position on SA-508, Cl. 1 Nozzles**

- No evidence of a safety concern associated with SA-508, Cl. 1 nozzle forging low initial USE
  - High USE values are generally anticipated
  - Conservative assessment shows that initial USE values need be only
     [[ ]] (more realistic estimates would place initial USE values needed at less than [[ ]])
  - Although not anticipated, failure of a small bore nozzle is accommodated within the plant design basis and is a part of the rationale associated with NRC acceptance of the conclusions related to aging management of RPV penetrations in BWRVIP-49-A
- Many plants have initial USE values that meet the requirement of 50 ft-lbs, but are based on low shear data
  - Efforts made to "justify" end of life USE values associated with these data are not needed to assure safe operation
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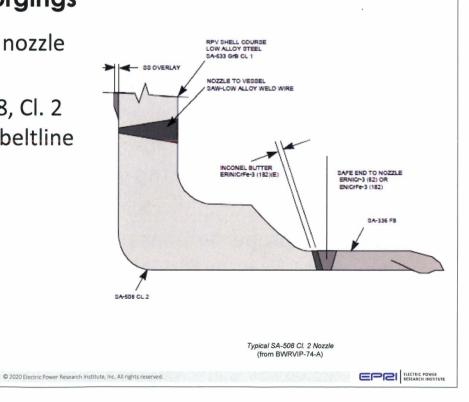
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# SA-508 Cl. 2 Nozzle Forgings

- Full penetration welded nozzle forgings
- Many BWRs have SA-508, Cl. 2 nozzles in the extended beltline
- Documented in LRAs

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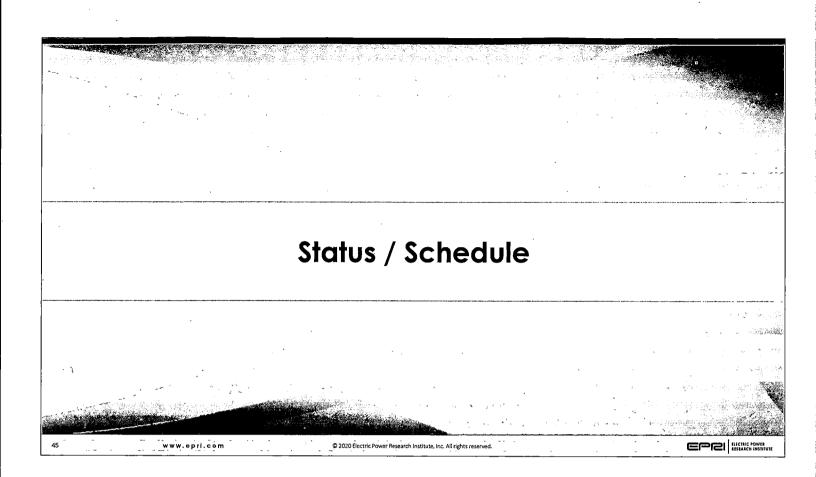
#### SA-508 Class 2 Initial USE Data Data analyzed from SA-508 Class 2 BWR forgings database. Charpy data for transverse and longitudinal specimens with high % shear. Transverse USE ( [] 11) – Mean = [[ ]], St Dev (σ) = [[ 11 **Content Deleted EPRI** Proprietary Information – Min measurement = [] 11 - Mean- $2\sigma = [[$ 11 - Mean- $3\sigma = [[$ 11 Minimum required initial USE = [[ ]] to attain EOL USE of 50 ft-lb, assuming a conservative upper-end fluence associated with 80-yr operations and best estimate %Cu (based on BWRVIP-173-A). If plant-specific fluence is lower than assumed in this analysis, likelihood of low EOL USE is very small, and no additional evaluation is needed Some plants have limited charpy data that does not include data indicative of upper shelf properties. These plants may need to rely on this evaluation as a basis for disposition. www.eprl.com © 2020 Electric Power Research Institute, Inc. All rights reserved

### Discussion

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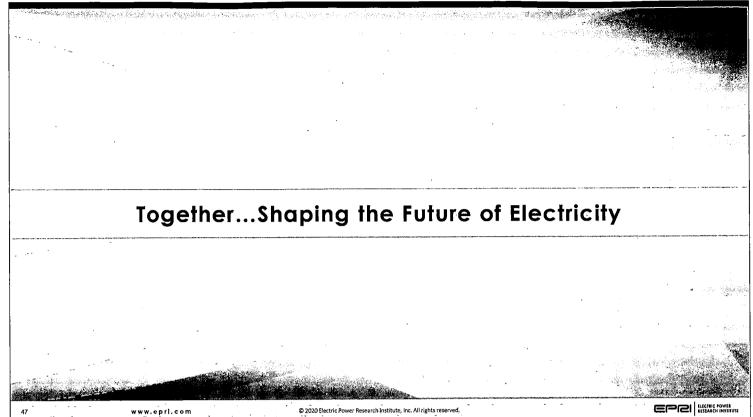
- No evidence of a safety concern associated with SA-508, Cl. 2 nozzle forging low initial USE
- Statistically improbable that nozzle end-of-life USE is less than 50 ft-lbs so long as nozzle end-of-life fluence is less than the upper end fluence assumed in the evaluation ( [[ Content Deleted EPRI Proprietary Information ]] )
- Some plants having only charpy data that are not indicative of upper shelf properties may potentially need to rely on this evaluation as a basis for disposition of nozzle forging TLAAs

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# BWRVIP-316 – Status & Future Plans

- EPRI Report 3002012536, BWRVIP-316: BWR Vessel and Internals Project Reactor Pressure Vessel Aging Management Evaluation for Extended Operations
  - To be published July 2020
  - BWRVIP intends to submit BWRVIP-316 to NRC for review and approval in September 2020



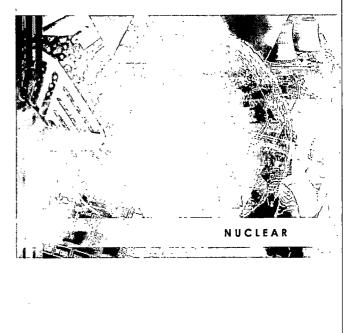


#### BWRVIP-316: BWR Reactor Pressure Vessel Aging Management for Extended Operations (OPEN SESSION)

Wayne Lunceford, P.E. Technical Executive, EPRI BWRVIP

NRC Public Meeting March 25, 2020

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### **BWRVIP** History

- Program created in 1994
- Initial objective to address IGSCC identified in BWR core shrouds
- Expanded to encompass all materials issues associated with BWR RPVs and internals
- Large body of knowledge currently, there are well over 300 BWRVIP reports, many of which have been revised one or more times
- Inspection and Flaw Evaluation (I&E) Guidance exists for all safety-related BWR RPV & internals components
- Consistent tracking, trending and evaluation of field inspection data for over 20 years
- Key involvement in development and deployment of IGSCC mitigation technologies
- History of proactive engagement with NRC to address aging management concerns associated with BWRs

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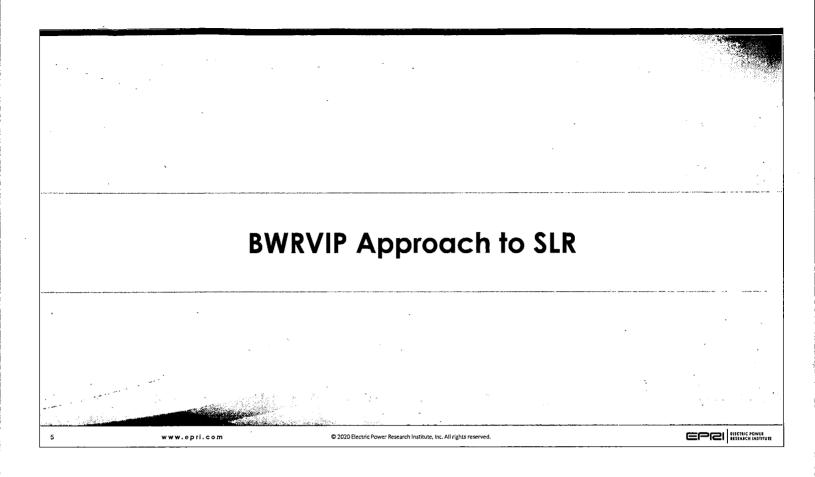
Degradati		I	Develop Developed	<b>A</b>	
0	Assessment	Inspection	Repair/Replace	Mitigation	
<u>Component</u>	(I&E) Guidelines	Guidelines	Design Criteria	Recommendations	
Core shroud	BWRVIP-76	BWRVIP-03	BWRVIP-02/-04	BWRVIP-62/-190	
Core spray	BWRVIP-18	BWRVIP-03	BWRVIP-16/-19/-34	N/A	
Shroud support	BWRVIP-38	BWRVIP-03	BWRVIP-52	BWRVIP-62/-190	
Top Guide	BWRVIP-26	BWRVIP-03	BWRVIP-50	N/A	
Core Plate	BWRVIP-25	BWRVIP-03	BWRVIP-50	BWRVIP-62/-190	
SLC	BWRVIP-27	BWRVIP-03	BWRVIP-53	BWRVIP-62/-190	
Jet pump assembly	BWRVIP-41	BWRVIP-03	BWRVIP-51	BWRVIP-62/-190	
CRD guide/stub tube	BWRVIP-47	BWRVIP-03	BWRVIP-17/-55/-58	BWRVIP-62/-190	
In-core housing/dry tube	BWRVIP-47	BWRVIP-03	BWRVIP-17/-55	BWRVIP-62/-190	
Instrument penetrations	BWRVIP-49	BWRVIP-03	BWRVIP-57	BWRVIP-62/-190	
LPCI coupling	BWRVIP-42	BWRVIP-03	BWRVIP-56	N/A	
Vessel ID brackets	BWRVIP-48	BWRVIP-03	BWRVIP-52	BWRVIP-62/-190	
Reactor pressure vessel	BWRVIP-74	N/A	N/A	BWRVIP-62/-190	
Primary system piping	<b>BWRVIP-75</b>	N/A	N/A	BWRVIP-62/-190	
Steam dryer	BWRVIP-139	BWRVIP-03	BWRVIP-181	N/A	
Access hole cover	BWRVIP-180	BWRVIP-03	TBD	BWRVIP-62/-190	
Top guide grid beam	BWRVIP-183	BWRVIP-03	BWRVIP-50	N/A	
Bottom head drain line	BWRVIP-205	N/A	BWRVIP-208	N/A	

# **Utility Participation & Commitment**

- On May 30, 1997 BWRVIP issued a letter to NRC, committing all BWR U.S. utilities to full support in the development, maintenance and implementation of the BWRVIP program
- This commitment stayed in place until superseded by NEI 03-08, Guideline for the Management of Materials Issues
  - NEI 03-08 Materials Initiative Policy Statement:

"... the industry will ensure that its management of materials degradation and aging is *forward-looking and coordinated* to the maximum extent practical. Additionally, the industry will *continue to* rapidly identify, react and *effectively respond to emerging issues*. The associated work will be managed to emphasize safety and operational risk significance as the first priority, appropriately balancing long term aging management and cost as additional considerations. To that end, as issues are identified and as work is planned, the groups involved in funding, managing and providing program oversight will ensure that the *safety and operational risk significance of each issue is fully established prior to final disposition*."

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# BWRVIP Approach to SLR (1 of 2)

- Existing aging management guidance is robust and, unless impacted by timedependent factors, remains adequate to manage the effects of aging
  - Although unanticipated adverse trends are possible, the BWRVIP program already includes activities to assess operating experience and new R&D results.
  - Field performance data are continually evaluated and aging management guidance updated as appropriate
- Time-Dependent Factors:
  - Factors associated with age-related degradation that directly or indirectly correlate with total accumulated operating time
  - Examples neutron fluence, fatigue cycles
- Many of the elements of an effective AMP are clearly independent of operating time (e.g., program scope, administrative controls, corrective actions, operating experience evaluation)
  - No need to revisit these program elements for SLR

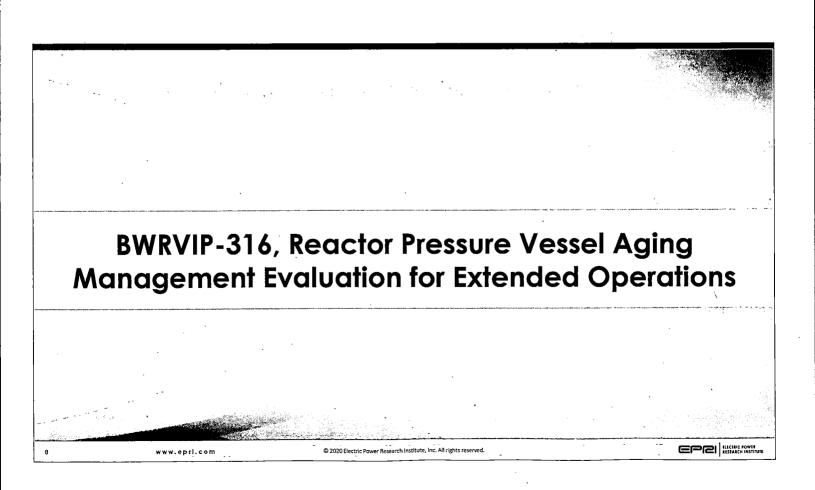
# BWRVIP Approach to SLR (2 of 2)

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- Aging management guidance implementation should be linked to engineering-based parameters
  - Contained in the underlying analytical work forming the technical basis for AMP implementation
  - Limitations on guidance applicability tied to parameters related to onset / progression of age-related degradation (*not operating period*)
- Applicability can be confirmed on a plant-specific basis (results need not be "bounding" for the entire fleet)
  - Each owner / licensee confirms that their plant satisfies the conditions for use of the guidance

- BWRVIP focus placed on technical aspects of aging management
- Decisions regarding plant operating period and licensing approach are the responsibility of the owner / licensee

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#### BWRVIP-316, RPV Aging Management Evaluation for Extended Operations

- Comprehensive review of the technical bases for conclusions regarding the impact of aging effects and associated degradation mechanisms on aging management of BWR RPV components for extended operations
- Review of operating experience relevant to aging management of BWR RPVs
- BWRVIP Evaluations Addressing RPV TLAAs
  - Fluence calculations
  - RPV Material Surveillance / Integrated Surveillance Program (ISP)
  - Upper shelf energy (USE)

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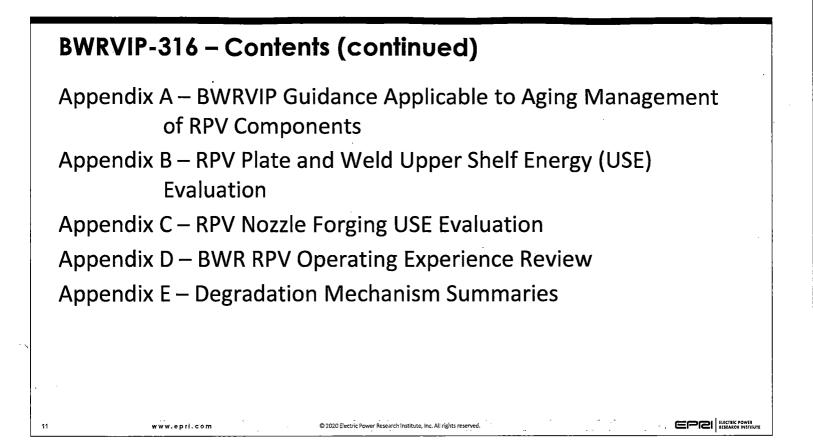
Represents an augmentation of BWRVIP-74-A, BWR RPV Inspection and Flaw Evaluation Guidelines for License Renewal

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**BWRVIP-316 - Contents** 

- 1. Introduction
- 2. Evaluation Scope
- 3. Degradation Assessment
- 4. Aging Management Review For Extended Operations
- 5. BWRVIP Evaluations Supporting RPV TLAAs
  - Inservice Inspection of RPV Circumferential Welds
  - USE Evaluation for RPV Plates and Welds
  - USE Evaluation for RPV Nozzle Forgings
- 6. AMP Attribute Assessment
  - Reviews the applicability of AMPs based on BWRVIP guidance that are credited by NUREG 2191 to manage aging of RPV components:
    - XI.M4, "BWR Vessel ID Attachment Welds"
    - XI.M7, "BWR Stress Corrosion Cracking"
    - XI.M8, "BWR Penetrations"
    - XI.M31, "Reactor Vessel Material Surveillance"
- 7. Summary of Results

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# BWRVIP-316 – Summary of Conclusions (1 of 2)

- Neutron fluence and fatigue are the primary <u>time-dependent</u> factors affecting aging management of BWR RPV components
- Management of SCC is not time-dependent

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- Detailed review of BWR RPV fleet operating experience does not show an adverse trend in performance
- Existing AMPs (ASME Section XI, relevant BWRVIP I&E Guidelines) remain adequate to address cracking due to SCC
- BWRVIP-06 Rev. 1-A safety assessment conclusions and BWRVIP inspection and flaw evaluation (I&E) guidance relevant to RPV aging management are not impacted by consideration of extended operations

# BWRVIP-316 – Summary of Conclusions (2 of 2)

- BWRVIP evaluations that generically support neutron embrittlement TLAA dispositions should be re-evaluated to address extended operations
  - RPV circumferential weld inservice inspection TLAAs are addressed by demonstrating that the acceptance criteria in BWRVIP-329 remain satisfied through plant EOL
  - RPV plate and weld upper shelf energy TLAAs are addressed by demonstrating that the acceptance criteria Appendix B of BWRVIP-316 are met
  - RPV nozzle upper shelf energy TLAAs are addressed by Appendix C of BWRVIP-316

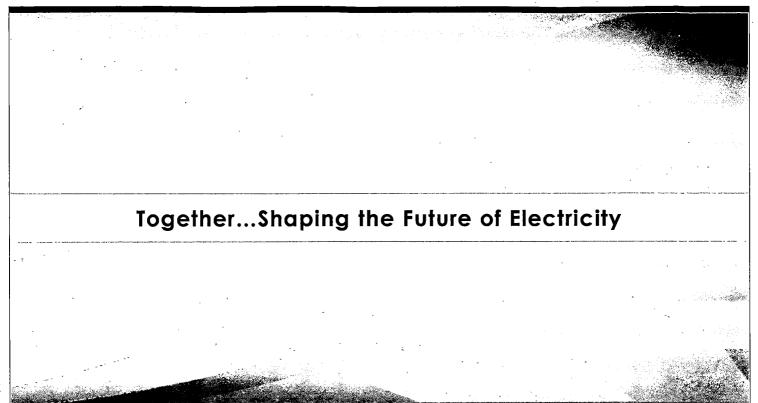
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#### BWRVIP-316 – Status & Future Plans

- EPRI Report 3002012536, BWRVIP-316: BWR Vessel and Internals Project Reactor Pressure Vessel Aging Management Evaluation for Extended Operations
  - To be published July 2020

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BWRVIP intends to submit BWRVIP-316 to NRC for review and approval in September 2020



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