



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION II
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200
ATLANTA, GEORGIA 30303-1200

March 19, 2020

Mr. James Barstow
Vice President, Nuclear Regulatory Affairs and Support Services Licensing
Tennessee Valley Authority
1101 Market Street, LP 4A-C
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2 – NRC INSPECTION REPORT
05000327/2020011 AND 05000328/2020011

Dear Mr. Barstow:

On February 14, 2020, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Sequoyah Nuclear Plant, Units 1 and 2 and discussed the results of this inspection with Mr. Scott Honeywell and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

Binoy B. Desai, Chief
Engineering Branch 3
Division of Reactor Safety

Docket Nos. 05000327 and 05000328
License Nos. DPR-77 and DPR-79

Enclosure:
Inspection Report

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SUBJECT: SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2 – NRC INSPECTION REPORT
05000327/2020011 AND 05000328/2020011 dated March 19, 2020

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ADAMS ACCESSION NUMBER: **ML20079J317**

<input type="checkbox"/> SUNSI Review		<input type="checkbox"/> Non-Sensitive <input type="checkbox"/> Sensitive		<input type="checkbox"/> Publicly Available <input type="checkbox"/> Non-Publicly Available		
OFFICE	DRS/EB3	DRS/EB3	DRS/EB3	DRS/EB3	DRS/EB3	DRS/EB3
NAME	S. Downey	I. Anchondo-Lopez	A. Butcavage	B. Collins	P. Cooper	B. Desai
DATE	3/ 18 /2020	03 /19 /2020	03/18 / 2020	03/ 17 /2020	03/ 18 /2020	03/ 19/2020

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U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Numbers: 05000327 and 05000328

License Numbers: DPR-77 and DPR-79

Report Numbers: 05000327/2020011 and 05000328/2020011

Enterprise Identifier: I-2020-011-0022

Licensee: Tennessee Valley Authority

Facility: Sequoyah Nuclear Plant, Units 1 and 2

Location: Soddy Daisy, TN

Inspection Dates: January 27, 2020 to February 14, 2020

Inspectors: S. Downey, Senior Reactor Inspector (Lead)
I. Anchondo-Lopez, Reactor Inspector
A. Butcavage, Reactor Inspector
B. Collins, Senior Reactor Inspector
P. Cooper, Reactor Inspector

Approved By: Binoy B. Desai, Chief
Engineering Branch 3
Division of Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a NRC inspection at Sequoyah Nuclear Plant, Units 1 and 2, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

OTHER ACTIVITIES – TEMPORARY INSTRUCTIONS, INFREQUENT AND ABNORMAL

71003 - Post-Approval Site Inspection for License Renewal

From January 27, 2020 to February 14, 2020, the NRC continued monitoring TVA's performance at Sequoyah by conducting a Post-Approval Site Inspection for License Renewal - Phase II in accordance with the license renewal inspection program (LRIP). This inspection took place prior to the period of extended operation (PEO) for Units 1 and 2, which will begin on September 17, 2020 and September 15, 2021, respectively. Per Inspection Manual Chapter 2516, the LRIP is the process used by NRC staff to verify the adequacy of aging management programs (AMPs), Time Limited Aging Analyses (TLAAs), and other activities associated with an applicant's request to renew an operating license of a commercial nuclear power plant beyond the initial licensing period under 10 CFR Part 54, "Requirements for the Renewal of Operating Licenses for Nuclear Power Plants."

Post-Approval Site Inspection for License Renewal (4 Samples)

- (1) License Conditions and Commitments for License Renewal, Implementation of Aging Management Programs and Time-Limited Aging Analyses

There are a total of 45 regulatory commitments linked to the renewed operating license for Sequoyah Nuclear Plant Units 1 and 2, issued in September 2015, which include the following: 26 commitments to enhance existing AMPs; 13 commitments to

implement new AMPs; 4 commitments to implement existing AMPs; and 2 standalone commitments (not associated with an AMP or TLAA). The inspectors reviewed a majority (greater than 70%) of regulatory commitments and associated AMPs and TLAAs in order to assess the adequacy and effectiveness of the license renewal program. For each item selected in the sample, the inspectors also reviewed the applicable license renewal program documents, conducted interviews with licensee staff, and performed system walkdowns to verify that the licensee completed the actions necessary to comply with the conditions listed in the renewed facility operating license. For those license renewal action items that were not completed at the time of this inspection, the inspectors verified, when possible, that there was reasonable assurance that such action items were on track for completion prior to the PEO or in accordance with an established implementation schedule consistent with the license renewal application (LRA), the NRC safety evaluation report (SER), and the Updated Final Safety Analysis Report (UFSAR) supplement. Commitment Items with pending actions requiring further inspection by the NRC are described in the Inspection Results section of this report.

The regulatory commitments selected for the inspection sample are listed below along with the completion status of the item at the time of the inspection. The full description of each commitment is available in the UFSAR supplement for license renewal, as revised, and Appendix A of the NRC SER issued as NUREG-2181 (ML15187A206).

- Commitment Item 1: Implementation of, and enhancements to, the Aboveground Metallic Tanks Program. Status: Complete
- Commitment Item 2: Enhancements to the Bolting Integrity Program. Status: Complete.
- Commitment Item 3: Implementation of, and enhancements to, the Buried and Underground Piping and Tanks Inspection Program. Status: Incomplete, some license renewal action items have not been completed. Specifically, the final installation and activation of the cathodic protection has not been completed and is expected to be implemented prior to entering the PEO. Based on the review of licensee actions completed at the time of this inspection, the timeliness of those actions, and the administrative controls in place to track the pending actions, the inspectors determined that there was reasonable assurance that the licensee would complete the necessary actions to meet Commitment #3. Therefore, no follow-up inspection is warranted.
- Commitment Item 4: Enhancements to the Compressed Air Monitoring Program. Status: Complete.
- Commitment Item 5: Enhancements to the Diesel Fuel Oil Monitoring Program: Status: Incomplete, one license renewal action item has not been completed. Specifically, the licensee has not put administrative controls in place to ensure that Commitment 5.C will be met during the PEO. To ensure the timely and technically acceptable completion of the remaining activities, Commitment Item 5 will remain open, is subject to further NRC inspection, and is presented as an observation in the Inspection Results section of this report.

- Commitment Item 6: Enhancements to the External Surfaces Monitoring Program. Status: Incomplete, some license renewal action items have not been completed. Specifically, the licensee has not put administrative controls in place to ensure that Commitment 6.A will be met during the PEO. To ensure the timely and technically acceptable completion of the remaining activities, Commitment Item 6 will remain open, is subject to further NRC inspection, and is presented as an observation in the Inspection Results section of this report
- Commitment Item 7: Enhancements to the Fatigue Monitoring Program and implantation of associated Time-limited Aging Analyses. Status: Complete.
- Commitment Item 8: Enhancements to the Fire Protection Program. Status: Complete.
- Commitment Item 9: Implementation of the Fire Water System Program. Status: Complete
- Commitment Item 10: Enhancements to the Flow Accelerated Corrosion Program. Status: Complete.
- Commitment Item 11: Enhancements to the Flux Thimble Tube Inspection Program. Status: Complete.
- Commitment Item 12: Enhancements to the Inservice Inspection - IWF Program. Status: Complete.
- Commitment Item 13: Enhancements to the Inspection of Overhead Heavy Load and Light Load Handling Systems. Status: Complete.
- Commitment Item 14: Implementation of, and enhancements to, the Internal Surfaces and Miscellaneous Piping and Ducting Components Program. Status: Complete.
- Commitment Item 15: Implementation of the Metal Enclosed Bus Program. Status: Complete.
- Commitment Item 16: Enhancements to the Neutron Absorbing Material Monitoring Program. Status: Complete
- Commitment Item 17: Implementation of the Non-EQ Cable Connections Program. Status: Complete.
- Commitment Item 18: Implementation of the Non-EQ Inaccessible Power Cables (400V to 35kV) Program. Status: Complete.
- Commitment Item 19: Implementation of the Non-EQ Instrumentation Circuits Test Review Program. Status: Complete.

- Commitment Item 22: Implementation of the One-Time Inspection Program. Status: Complete.
- Commitment Item 23: Implementation of the One-Time Inspection – Small Bore Piping Program. Status: Complete.
- Commitment Item 24: Enhancements to the Periodic Surveillance and Preventative Maintenance Program. Status: Complete.
- Commitment Item 25: Enhancement to the Protective Coating Program. Status: Complete.
- Commitment Item 26: Enhancement to the Reactor Head Closure Studs Program. Status: Complete
- Commitment Item 27: Enhancements to the Reactor Vessel Internals Program. Status: Complete.
- Commitment Item 28: Enhancements to the Reactor Vessel Surveillance Program
- Commitment Item 29: Implementation of the Selective Leaching Program. Status: Incomplete, some license renewal action items have not been completed. Specifically, there were eight valves in the High Pressure Fire Protection system made of gray cast iron and were therefore in the scope of the Selective Leaching Program. However, the valves were installed in 1998 and inspecting them prior to 35 years of service would not provide a true indication of whether selective leaching was occurring. This issue was documented in both the licensee's FSAR and Selective Leaching Program Basis Document, and preventive maintenance tasks were scheduled to occur between 2033 and 2038. On this basis, the inspectors determined that there are appropriate administrative controls in place to ensure that Commitment 29 will be met. Therefore, no follow-up inspection is warranted.
- Commitment Item 30: Enhancement to the Steam Generator Integrity Program concerning tube plugging material. Status: Complete
- Commitment Item 31: Enhancements to the Structures Monitoring. Status: Complete.
- Commitment Item 32: Implementation of the Thermal Aging Embrittlement of Cast Austenitic Stainless Steel (CASS) Program. Status: Complete.
- Commitment Item 33: Enhancements to the Water Chemistry Control– Closed Treated Water Systems Program. Status: One license renewal action item has not been completed. Specifically, the licensee has not put administrative controls in place to ensure that Commitment Item 33.D will be met during the PEO. To ensure the timely and technically acceptable completion of the remaining activities, Commitment Item 33 will remain open, is subject to

further NRC inspection, and is presented as an observation in the Inspection Results section of this report

- Commitment Item 34: Enhancement to the Containment Leak Rate Program. Status: Complete.
- Commitment Item 35: Enhancements to Containment Inservice Inspection – IWE Program made in response to RAI B.1.6-1 (MLA13190A276, ML13312A005, ML13294A462, ML14058A131). Status: Complete
- Commitment Item 36: Enhancements to the Inservice Inspection Program. Status: One license renewal action item has not been completed. Specifically, the licensee is unable to meet Commitment 36.A as written because there is currently no ASME or EPRI qualified technique available to detect cracking in CASS components. The issue has been entered into the licensee's Corrective Action Program as CR 1582890. Under this CR, the licensee will track industry progress on developing qualified examination techniques. Once a technique is qualified, the ISI program will be revised to incorporate the inspections described in the commitment. On this basis, the inspectors determined that there are appropriate administrative controls in place to ensure that Commitment 36.A will be met. Therefore, no follow-up inspection is warranted.
- Commitment 37: Implement OE and AMPs in accordance with TVA response to RAI B.0.4-1 (ML13213A027) and 10/17/13 letter, RAIs B.0.4-1 and A.1-1a (ML13294A462). Status: Complete.
- Commitment Item 38: Enhancements to the Service Water Integrity Program. Status: Complete.
- Commitment Item 41: Implementation of the Masonry Wall Program. Status: Complete.
- Commitment Item 44: Implementation of the RG 1.127, Inspection of Water-Control Structures Associated with Nuclear Power Plants Program. Status: Complete for Units 1 and 2. The inspection of the submerged portion of the water-controlled structures has not been performed, however, the requirement to perform the inspection has been incorporated into program documents. Based on the review of licensee actions completed at the time of this inspection, the timeliness of those actions, and the administrative controls in place to track the pending actions, the inspectors determined that there was reasonable assurance that the licensee would complete the necessary actions to meet Commitment 44. Therefore, no follow-up inspection is warranted.
- Commitment Item 45: Implementation of the 161-kV Oil Filled Cables Program. Status: Complete.

(2) Newly Identified Structures, Systems, and Components (SSCs)

Since the renewed license was issued, the licensee has performed two assessments to identify any SSCs that would have been subject to an aging management review or evaluation as a time-limited aging analysis in accordance with 10 CFR 54.21. The inspectors reviewed the licensee's assessments and noted that the following SSCs were identified as being within the scope of license renewal and subject to aging management review:

1. Auxiliary Essential Raw Cooling Water (AERCW) Pump Structure
2. Essential Raw Cooling Water Valve Boxes 1, 3, 4, 5, and 6, and Thermometer Boxes 1 and 2
3. High Pressure Fire Protection Inaccessible Cables
4. Main Feedwater Control Mechanical Components

For each newly identified SSC, the inspectors determined that the licensee took appropriate actions in accordance with 10 CFR 54.37(b). Specifically, the inspectors verified that the SSCs have been included in the FSAR update required by 10 CFR 50.71(e), and that the FSAR update described how the effects of aging are managed such that the intended function(s) in 10 CFR 54.4(b) will be effectively maintained during the period of extended operation. The inspectors also reviewed a list of plant modifications performed from the time the LRA was submitted to the time the renewed operating license was issued, to identify any potentially new SSCs that would have been subject to aging management review at the time the NRC was reviewing the LRA. Based on this review, the inspectors did not identify any new SSCs that were subject to the provisions of 10 CFR 54.37(b).

(3) Description of Aging Management Programs in the UFSAR Supplement

As part of the review of implementation activities for the selected AMPs, the inspectors reviewed the corresponding UFSAR sections to verify that the program descriptions were consistent with the license renewal application and the corresponding section of the NRC safety evaluation report. The inspectors reviewed three versions of the UFSAR supplement for license renewal as follows:

- The inspectors reviewed the UFSAR supplement submitted with the LRA, as revised, to identify the program attributes and future inspection activities that were originally relied upon for the approval of the renewed operating license.
- The inspectors reviewed the revision of the UFSAR submitted to the NRC pursuant to the requirements in 10 CFR 50.71(e)(4) following the issuance of the renewed operating license to verify that the UFSAR supplement for license renewal was included as required by the condition of the renewed operating license.
- The inspectors reviewed the latest revision of the UFSAR supplement for license renewal (Amendment 28) to verify that the program attributes and inspection activities were consistent with the AMPs as originally approved by the NRC and subsequent revisions performed under the provisions of 10 CFR 50.59. The inspectors also verified that any changes caused by the inclusion of newly identified SSCs were included in the UFSAR supplement.

(4) Changes to license renewal commitments and the UFSAR supplement for license renewal

As part of the review of license renewal commitments, AMPs, and TLAAs described in this report, the inspectors reviewed a sample of license renewal commitment change documents to verify the licensee followed the guidance in NEI 99-04, "Guidelines for Managing NRC Commitment Changes," for any change to the commitments, including their elimination. Except as identified in the Inspection Results section of the report, the inspectors verified that the licensee properly evaluated, reported, and approved where necessary, changes to license renewal commitments listed in the UFSAR in accordance with 10 CFR 50.59, as required by the Renewed Operating License (Section 2.C.(31))

INSPECTION RESULTS

Observation: Observation for Commitment Item 5 – Enhancements to the Diesel Fuel Oil Monitoring Program	71003
<p>In 2018, the licensee revised Commitment Item No. 5.C, UFSAR section 13.9.1.A.1.8, and UFSAR section 13.9.2, to remove the requirement to perform a ten-year periodic cleaning of the standby diesel fuel oil day tanks and the High Pressure Fire Protection (HPFP) diesel fuel oil storage tank. The revised commitment now requires only an inspection of the tanks to be performed at least once prior to the PEO and at succeeding ten-year intervals and states that these inspections will be supplemented by cleaning, to remove sediment, if warranted, based on the results of periodic cleaning of the fuel oil storage tanks and 7-day tanks, periodic cleaning of the fuel pump suction strainers and filters, and day tank sediment and particulate monitoring. While reviewing the licensee's commitment change process documents (i.e. commitment change evaluation form, 50.59 screen, SAR change request), the inspectors noted that no justification was provided for removing the requirement to periodically clean the HPFP diesel fuel oil storage tank. Specifically, all documents reviewed focus only on the 550-gallon diesel fuel oil day tanks and the licensee's physical inability to visually inspect and clean them due to their design and configuration. It is noted that the HPFP diesel fuel oil storage tank is of a different design than the day tanks and provides the access needed to perform a 100% visual inspection, which the licensee performed in May 2018 and found no sludge or deposits that required cleaning.</p> <p>During an interview with the licensee's staff regarding the commitment change, the inspectors determined that the additional activities in the revised commitment that would provide a basis for cleaning the tanks (i.e. cleaning 7 day tanks, cleaning strainers and filters, etc.) are also only applicable to the fuel oil day tanks. When asked what activities would form the basis for cleaning the HPFP diesel fuel oil storage tank, the licensee responded that cleaning of the tank would be performed based on the results of the ten-year periodic visual inspections. The inspectors noted that the licensee's response contrasted with what was stated in the commitment. Subsequently, the licensee informed the inspectors that the change to Commitment 5.C that removed the cleaning of the fuel oil day tanks inadvertently implied that the HPFP tank would also not be cleaned. This issue has been captured in the licensee's Corrective Action Program as CR 1583386.</p> <p>At the time of this inspection, this commitment item had not been completed and the licensee needed to take the necessary actions to address CR1583386. Therefore, Commitment Item 5 will require further NRC inspection to ensure timely and technically acceptable completion.</p>	

Observation: Observation for Commitment 6: Enhancements to the External Surfaces Monitoring Program	71003
<p>After reviewing the program procedures and interviewing the responsible plant personnel regarding this program, the inspectors identified that Commitment 6.A had not been adequately implemented. Specifically, Commitment 6.A requires that External Surfaces Monitoring Program procedures be revised to clarify that periodic inspection of systems within the scope of license renewal and subject to aging management review (AMR) will be performed. The commitment also states that the inspections shall include areas surrounding the subject systems to identify hazards to those systems and that inspections of nearby systems that could impact the subject systems will include SSCs that are in scope and subject to an AMR.</p> <p>The program basis document identifies procedure NPG-SPP-09.7.1, "Corrosion Control - General, Localized and Galvanic, and Stress Corrosion Cracking Program," as the implementing procedure of this commitment. The inspectors identified that procedure NPG-SPP-09.7.1 does not address the commitment's specific inspection actions but references procedure NPG-SPP-09.0.1.1, "System Monitoring and Trending" as the implementing procedure that will "define the engineering requirements for performing system walk-downs for determining the material condition of systems including documenting adverse conditions such as material wastage due to corrosion." Upon reviewing procedure NPG-SPP-09.0.1.1, the inspectors determined that the procedure did not include the specific actions needed to address Commitment 6.A. This issue was captured in the licensee's Corrective Action Program as CR 1586277.</p> <p>At the time of this inspection, this commitment item had not been completed and the licensee needed to take the necessary actions to address CR1586277. Therefore, Commitment Item 6 will require further NRC inspection to ensure timely and technically acceptable completion.</p>	

Observation: Observation related to the Clarity of Acceptance Criteria in the Preventative Surveillance and Preventative Maintenance Program	71003
<p>As documented in the NRC SER (ML15187A206 - NUREG-2181, pg 3-204), RAI B.1.31-4 was issued to request that the licensee clarify whether acceptance criteria would permit degradation by verifying the absence of an aging effect or whether it would permit degradation by verifying that no unacceptable aging effect exists. In response, the licensee stated that they had revised PSPM Program Element 6 by deleting the discussion of "significant changes in material properties" and "unacceptable loss of material," and inserting a new sentence stating that "any indication or relevant condition of degradation detected is evaluated." The inspectors noted that the staff found the licensee's response to RAI B.1.31-4 acceptable because the acceptance criteria were clarified to ensure that any relevant condition is evaluated, which is consistent to verifying the absence of an aging effect.</p> <p>While reviewing the PSPM program, the inspectors identified that the original language that prompted the issuance of RAI B.1.31-4 was still present in the program basis document as well as in several implementing procedures. The inspectors also noted several examples where the licensee used similar unclear acceptance criteria, such as "no unacceptable cracking", "no significant corrosion that would impede performance", and "no unacceptable loss of coating integrity", in PSPM program documents. When questioned about the acceptance criteria, the licensee acknowledged that, while clarifying changes were made to in response to RAI B.1.31-4, the original language still existed in several PSPM program documents. This issue has been entered into the licensee Corrective Action Program as CR</p>	

1586169. After further discussion with the inspectors, the licensee updated the CR to include an assessment of all of the acceptance criteria in the PSPM program.

Given the expansive nature of the PSPM, and the number of procedures and PMs affected, the NRC will need to perform follow-up inspection activities to ensure that the acceptance criteria have all been clarified as needed to ensure that the effects of aging are adequately managed.

Observation: Observation for Commitment Item 33: Enhancements to the Water Chemistry Control – Closed Treated Water Systems Program	71003
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Commitment 33.D requires the licensee to revise the Water Chemistry Control - Closed Treated Water Systems Program procedures to perform sampling and analysis of the glycol cooling system per industry standards and in no case greater than quarterly unless justified with an additional analysis. After reviewing the program procedures and interviewing the responsible plant personnel regarding the program, the inspectors identified that the wrong procedures were revised to implement Commitment 33.D. Specifically, the program basis document states that Commitment 33.D is implemented with the issue of procedures CHEM-003, "Closed Cooling Water Chemistry Strategic Plan", 0-PI-CEM-000-001.0, "Periodic Chemistry Requirements for Security Backup Diesel Generator and High Pressure Fire Protection Diesel Pump Engine Coolant", and 0-TI-CEM-000-001.4, "Auxiliary System and Common System Chemistry Specifications". All three procedures identify the High Pressure Fire Protection (HPFP) Diesel Pump Cooling Water as the point at which glycol will be sampled quarterly to meet the commitment, while CHEM-003 and 0-TI-CEM-000-001.4 both also identify the Security Backup Diesel Generator (SBDG) Cooling Water as a sampling point to meet the commitment. However, the inspectors noted that the glycol cooling system is part of the Ice Condenser System and not the systems referenced in the implementing procedures. When asked for clarification on the sampling point(s) that are applicable to Commitment 33.D, the licensee responded that the two applicable sampling points are listed in PM 063606271, which sample the Ice Condenser System from the main header at two locations on an 18-month frequency, not quarterly as required to meet the commitment. This issue was captured in the licensee's Corrective Action Program as CR 1583096.

At the time of this inspection, this commitment item had not been completed and the licensee needed to take the necessary actions to address CR1583096. Therefore, Commitment Item 33 will require further NRC inspection to ensure timely and technically acceptable completion.

Observation: Observation of Incorrect Categorization of Quality Procedures	71003
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During the review of the External Surface Condition Monitoring program, the inspectors observed that the implementing periodic instructions were classified as Non-Quality Related documents. The inspectors reviewed procedure TVA-NQA-PLN89-A, "Nuclear Quality Assurance Plan (NQAP)" and questioned the licensee's classification of the periodic instructions. As a result, the licensee performed a review of all license renewal periodic instructions and identified that seven instructions had been incorrectly classified as "Non-Quality Related" affecting a total of six aging management programs. The licensee placed this issue into their corrective action program as condition report CR 1585902. Since the classification of the procedures is part of the implementation of the site NAQP, and not a regulatory commitment for license renewal, no follow-up inspection is warranted.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On February 14, 2020, the inspectors presented the NRC inspection results to Mr. Scott Honeywell and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71003	Calculations	CEB-GEN-88PR01	Pipe Rupture Evaluation of Wear on Thimble Tubes and Acceptance Criteria	Rev. 2
		CN-MRCDA-08-2	CRDM Head Adapter Wear Justification	Rev. 1
		SI File No. 1901378.301	Evaluation of Observed Thinning within 30"x24" ERCW Branch Piping Tee	11/26/2019
	Corrective Action Documents	Condition Reports	16684, 684461, 684468, 810761, 991942, 993922, 1236019, 11334236, 1379157, 1407748, 1408833, 1417967, 1442339, 1467388, 1532825, 1561562, 1582509, 1582605, 1582764, 1582854, 1582858, 1582890, 1583050, 1583096, 1583351, 1583386, 1583653, 1585902, 1585757, 1585789, 1585902, 1586060, 1586169, 1586277, 1586388, 1586389, 1586461	
	Drawings	1,2 -47W450-6	SEQ Nuclear, Mechanical Essential Raw Cooling Water	Rev.15
		10005D62	Straight CEW Tube Plug, Steam Generator	Rev. 2
		ISI-0298-C-01	173" PWR Vessel General Arrangement	Rev. 3
		ISI-0298-C-02	Reactor Vessel Seam Welds	Rev. 3
		ISI-0298-C-03	Reactor Vessel Interior Seam Welds	Rev. 3
		ISI-0298-C-04	Reactor Vessel Support Locations	Rev. 2
		ISI-0298-C-05	Reactor Vessel Core Support Structure	Rev. 0
	Engineering Changes	28-31	SAR Change Package: Revise PSPM Program to Replace Diesel Generator Starting Air Aluminum Valves rather than Inspect	7-16-18
		EWR No.18-EPG-067-199	Summary of Flow Accelerated Corrosion / Erosion Program inspection for SQN License Renewal Commitments 114446167 & 114446284 in accordance with NPG-SPP-09.0 Section 3.2.9C.	9/29/18
		PMSICHG-SQN-TICKET-1281317	TVA PM Change/Deferral: PM #120115 (Selective Leaching Inspections)	07/22/2019
	Engineering Evaluations	1400627.401	Flaw Tolerance Evaluation of Sequoyah Units 1 and 2 CASS Components	Rev. 1
		1400627.403	Development of a CASS Aging Management Program for Sequoyah Unit 1 & 2	Rev. 1
		CN-RIDA-12-39	Sequoyah Units 1 & 2 Lower Internals Hold Down Capacity	Rev. 0

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Evaluation For Hold-down Spring Height Measurement	
		CN-RIDA-18-7	Sequoyah Units 1 & 2, Lower Internals Hold-Down Spring Capability Evaluation for Fuel Transition	7/26/18
		EWB No. 17-SIE-018-205	SQN Emergency Diesel Generator (EDG) 550 Gallon Fuel Oil Day Tank License Renewal Justification	11/07/2017
		FS1-0037703-1.0	Summary of Evaluation Results of Degraded Baffle Bolting on HTP Fuel at Sequoyah Unit 1, RFO-22	5/3/2018
		G-55	General Engineering Specification for Technical and Programmatic Requirements for the Protective Coating Program for TVA Nuclear Plants	Rev. 24
		LTR-AMLR-18-30 (EDMS No. B88 180514 800)	Sequoyah Unit-1, Reactor Vessel Baffle Former Bolt Acceptable Bolting Pattern As-Found Real Time Analysis Summary	5/14/2018
		LTR-AMLR-18-72	Sequoyah Unit-2, Reactor Vessel Baffle Former Bolt Ultrasonic Inspection Summary Report	11/27/18
		PM-120115-SQN	PM #120115 (Selective Leaching Inspections) Evaluation	07/23/2019
		Sequoyah Unit 1 Cycle 23	Thimble Tube Damage (Trending) Table , Cycle 16 thru 23	Oct. 2019
		Sequoyah Unit 2, Cycle 22	Thimble Tube Damage (Trending) Table, Cycle 11 thru 22	Nov. 2019
		TVA065-REPT-062	TVA, Sequoyah nuclear Plant, Application of Component Specific License Renewal Scoping Requirements to Components with Level III Coatings	Rev. 0
	Miscellaneous	0-MI-EPM-079-049.0	Fuel Transfer Equipment Checkout (System 79)	Rev. 28
		0-MI-MRR-068-005.0	Sequoyah Nuclear Plant, Unit 1 & 2, Maintenance Instruction, Removal of Reactor Pressure Vessel Head and Attachments	Rev. 0065
		0-MI-MRR-068-006.0	Sequoyah Nuclear Plant, Unit 1 & 2, Maintenance Instruction, Removal of Reactor Pressure Vessel Head and Attachments	Rev. 0057
		0-MI-MXX-000-009.2	Routine Maintenance Involving Unistrut Clamps, Anchored Connections, Instrument Bolting, and Miscellaneous Bolting	Rev. 6
		0-MI-MXX-068-005.0	Steam Generator Primary Side Maintenance Activities	Rev. 0027
		0-PI-DXI-000-	ASME Section XI ISI/NDE Program, Unit 1 and Unit 2	Rev. 0002

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		114.4		
		0-PI-DXI-000-116.3	ASME Section XI IWE/IWL Containment Inservice Inspection (CISI) Program	Rev.0
		0-PI-DXX-000-100.06.1	Insulated Component External Surfaces Inspection	Rev. 0
		0-PI-DXX-000-100.37.1	License Renewal Operating Experience (OE), Review Instruction	Rev. 0000
		0-PI-DXX-000-100.6.2	Uninsulated Component External Surfaces Inspection	Rev. 0
		0-PI-DXX-000-37.1	License Renewal Operating Experience (OE) Review Instruction	Rev. 0
		0-PI-DXX-000-704.1	MIC and Cavitation Degradation Monitoring Program	Rev. 0008
		0-PI-DXX-000-704.1	MIC and Cavitation Degradation Monitoring Program	Rev. 0008
		0-PI-DXX-000-750	Piping Inspection in Tunnels and Frequency	Rev. 0
		0-PI-NUC-000-009.0	Spent Fuel Pool Coupon Tree Surveillance	Rev, 0000
		0-SI-SXI-068-114.4	Sequoyah Nuclear Plant, Unit-0, Surveillance Instruction, Steam Generator Tubing Inservice Inspection and Augmented Inspections	8/10/2018
		0-TI-DXI-000-115-0	ASME Section XI Repair and Replacement Program	Rev. 10
		0-TI-DXX-000-001.0	Flow Accelerated Corrosion Program	Rev. 11
		0-TI-DXX-000-010.0	Sequoyah Nuclear Plant, Unit-0, Technical Instruction, Protective Coatings Program	Rev. 0004
		0-TI-DXX-000-016.0	Erosive Wear Degradation Monitoring Program	Rev. 0002
		0-TI-SXX-000-146.0	Program for Implementing NRC Generic Letter 89-13	Rev. 0007
		0-TI-XXX-000-704.0	MIC AND Cavitation Degradation Monitoring	Rev. 0009

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		0-TPP-DXX-00-100.22	License Renewal Aging Management Program Basis Document One-Time Inspection Program	Rev. 0
		0-TPP-DXX-000-100.01	License Renewal Aging Management Program Basis Document: Aboveground Metallic Tanks Program	Rev. 0000
		0-TPP-DXX-000-100.02	License Renewal Aging Management Program Basis Document: Bolting Integrity Program	Rev. 0000
		0-TPP-DXX-000-100.03	License Renewal Aging Management Program Basis Document Buried and Underground Piping and Tanks Inspection Program	Rev. 0
		0-TPP-DXX-000-100.04	License Renewal Aging Management Program Basis Document: Compressed Air Monitoring Program	Rev. 0001
		0-TPP-DXX-000-100.05	License Renewal Aging Management Program Basis Document, Diesel Fuel Monitoring Program	Rev. 0000
		0-TPP-DXX-000-100.06	License Renewal Aging Management Program Basis Document: External Surface Monitoring Program	Rev. 0001
		0-TPP-DXX-000-100.08	License Renewal Aging Management Program Basis Document, Fire Protection Program	Rev. 0001
		0-TPP-DXX-000-100.09	License Renewal Aging Management Program Basis Document, Fire Water System Program	Rev. 0000
		0-TPP-DXX-000-100.10	License Renewal Aging Management Program Basis Document: Flow-Accelerated Corrosion Program	Rev. 0000
		0-TPP-DXX-000-100.11	License Renewal Aging Management Program Basis Document, Flux Thimble Tube Inspection Program	Rev. 0001
		0-TPP-DXX-000-100.12	License Renewal Aging Management Program Basis Document, Inservice Inspection - IWF Program	Rev. 0001
		0-TPP-DXX-000-100.13	License Renewal Aging Management Program Basis Document Inspection of Overhead Heavy Load and Light Load (Related to Refueling) Handling Systems Program	Rev.0
		0-TPP-DXX-000-100.14	License Renewal Aging Management Program Basis Document: Internal Surfaces in Miscellaneous Piping and Ducting Components Program	Rev. 0000
		0-TPP-DXX-000-	License Renewal Aging Management Program Basis	Rev. 0001

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		100.15	Document: Metal Enclosed Bus Inspection Program	
		0-TPP-DXX-000-100.17	License Renewal Aging Management Program Basis Document, Non-EQ Cable Connections	Rev. 0000
		0-TPP-DXX-000-100.18	License Renewal Aging Management Program Basis Document Non-EQ Inaccessible Power Cables (400 V to 35 kV) Program	Rev.0
		0-TPP-DXX-000-100.19	License Renewal Aging Management Program Basis Document: Non-EQ Instrumentation Circuits Test Review	Rev. 0001
		0-TPP-DXX-000-100.21	License Renewal Aging Management Program Basis Document, Oil Analysis Program	Rev. 0001
		0-TPP-DXX-000-100.23	License Renewal Aging Management Program Basis Document: Small Bore Piping Program	Rev. 0000
		0-TPP-DXX-000-100.24	License Renewal Aging Management Program Basis Document: Periodic Surveillance and Preventative Maintenance Program	Rev. 0000
		0-TPP-DXX-000-100.25	License Renewal Aging Management Program Basis Document: Protective Coating Monitoring and Maintenance Program	Rev. 0001
		0-TPP-DXX-000-100.26	License Renewal Aging Management Program Basis Document, Reactor Head Closure Studs Program	Rev. 0000
		0-TPP-DXX-000-100.28	Sequoyah Nuclear Plant, Unit 0, Technical Program Procedures, License Renewal Aging Management Program Basis Document ,Reactor Vessel Surveillance Program	Rev.0001
		0-TPP-DXX-000-100.29	License Renewal Aging Management Program Basis Document: Selective Leaching Program	Rev. 0001
		0-TPP-DXX-000-100.30	Sequoyah Nuclear Plant, Unit 0, License Renewal Aging Management Program Basis Document, Steam Generator Integrity Program	Rev. 0000
		0-TPP-DXX-000-100.31	License Renewal Aging Management Program Basis Document Structures Monitoring Program	Rev. 0
		0-TPP-DXX-000-100.32	License Renewal Aging Management Program Basis Document: Thermal Renewal Aging Embrittlement of Cast Austenitic Stainless Steel (CASS) Program	Rev. 0001
		0-TPP-DXX-000-	License Renewal Aging Management Program Basis	Rev. 0000

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		100.33	Document, Closed Treated Water Program	
		0-TPP-DXX-000-100.35	License Renewal Aging Management Program Basis Document Containment Inservice Inspection – IWE Program	Rev.0
		0-TPP-DXX-000-100.36	License Renewal Aging Management Program Basis Document: Inservice Inspection Program	Rev. 0000
		0-TPP-DXX-000-100.38	License Renewal Aging Management Program Basis Document Service Water Integrity Program	Rev. 0000
		0-TPP-DXX-000-100.41	License Renewal Aging Management Program Basis Document Masonry Wall Program	Rev.0
		0-TPP-DXX-000-100.44	License Renewal Aging Management Program Basis Document RG 1.127, Inspection of Water Control Structures Associated with Nuclear Power Plants Program	Rev. 0
		0-TPP-DXX-000-100.45	License Renewal Aging Management Program Basis Document, 161-kV Oil-Filled Cable Program	Rev. 0000
		1-PI-RVI-000-301.0	Sequoyah Nuclear Plant, Unit 1, Periodic Instruction, PWR Reactor Vessel Internals Inspection Program	Rev. 0000
		17-EPG-999-201	Summary of Small Bore Piping One Time Inspection Program Inspections for SQN	12/28/2017
		17-EPG-999-202	Summary of Structures Monitoring Program Inspections Required for SQN License Renewal Commitment #114456053 and #114456121	2/12/18
		2-PI-RVI-000-301.0	Sequoyah Nuclear Plant, Unit 2, Periodic Instruction, PWR Reactor Vessel Internals Inspection Program	Rev.0000
		3002006813	Materials Reliability Program: Examination of Small Bore Socket Welds for Vibration and Thermal Fatigue	Rev. 1
		3025399	Destructive Examination of Socket Weld Indications	Rev. 1
		51-9199524-000	Technical Justification for Demonstrating the Capabilities of CRDM/CEDM Examination Procedures of Nozzle Bores with Thermal Sleeve Centering Tab Wear	Rev. 0
		AMP 070.220, LMS 50002384	Sequoyah Flux Thimble Tube Inspection Program, Job Familiarization Guide	5/18/2015
		ASME Section XI, Subsection IWF,	IWF-1300, Support and Examination Boundaries	2007

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		Article IWF-1000		
		BP-257	Integrated Materials Issues Management Plan	Rev. 0019
		CTD 114454762	Commitment Tracking Database, to Revise Reactor Closure Studs Program Procedures	3/29/2018
		CXV645J	Form 6 - Procurement Data Sheet, Plug, Tubing, Q Level: QA 1, ASME SEC III: CLASS 1, Straight CEW Tube Plugs, ASME SB-166 ALLOY 690, for Sequoyah Unit-1 & 2 Steam Generators (RSG)	Rev. 0
		DOWG 16-01	Resource Manual for IP-ENG-001 for IP-ENG-001, Standard Design Process	11/12/2018
		Emp. ID No. UJ86NTACG	Learning History Completion Records for AMP 070.220 Flux Thimble Tube Inspection Program	7/24/2017
		ESP-070.531	BFN, SQN, WBN, Corporate Engineering Personnel, TPD-ESP, Mentoring/Position Specific Guide, Raw Water Corrosion Inspection	Rev. 30
		ESP070.234	BFN, SQN, WBN, Corporate Engineering Personnel, TPD-ESP, Mentoring/Position Specific Guide, Protective Coating Program Engineer	Rev. 28
		EWR No. 17-SBE-026-199	Summary of Fire Water System Program Inspections for SQN License Renewal	Rev. 2
		EWR-17-EPG-317-171	SQN License Renewal Implementation One-Time inspection	9/19/17
		G-29-P.S.4.M.4.4	ASME Section III and Non-ASME Section III (Including AISC, ANSI, B31.1, and ANSI B31.5) Bolting Material	Rev. 7
		G-29B P.S.4.M.1.1	Material Fabrication and Handling Requirements for Austenitic Stainless Steel	Rev. 26
		G-55	General Engineering Specification, Technical and Programmatic Requirements for the Protective Coating Program for TVA Nuclear Plants	Rev. 24
		L18 140519 800	Approval of Areva Procedure 54-UT-901-003, "Automated Ultrasonic Examination of Wear in RPV Closure Head	05/19/2014

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			Thermal Sleeves"	
		LTR-CNL-15-095	Sequoyah Nuclear Plant - Revision to the Reactor Pressure Vessel Surveillance Withdrawal Schedule For License Renewal	5/14/15
		LTR-RIDA-07-176	Thermal Sleeve Wear Evaluation for Sequoyah Unit 1 & 2 and Watts Bar Unit 1	Rev. 1
		NCO 114446365	Commitment Tracking Database, Commitment Tracking Sheet to Revise Flux Tube Inspection Program	3/13/2018
		NPG-SPP-03.15	TVA NPG, Standard Programs and Processes Management, FSAR Management	Rev. 0002
		NPG-SPP-06.2	TVA, NPG Standard Programs and Processes, Preventive Maintenance	Rev. 13
		NPG-SPP-08.1	Nuclear Fuel Management, Att. 8, Spent Fuel Pool Neutron Absorber Material Monitoring Program	Rev. 0018
		NPG-SPP-09.0.6	Conduct of Engineering Programs	Rev. 0005
		NPG-SPP-09.1	ASME Code and Augmented Programs	Rev. 0010
		NPG-SPP-09.1.5	ASME Section XI Containment ISI Program	Rev.0
		NPG-SPP-09.1.7	Steam Generator Program	Rev. 0000
		NPG-SPP-09.14	TVA, NPG Standard Programs and Processes Generic Letter (GL) 89-13 Implementation	Rev. 0005
		NPG-SPP-09.15	Underground Piping and Tanks Integrity Program	Rev. 8
		NPG-SPP-09.26.16	TVA, NPG Standard Programs and Processes, PWR Reactor Vessel Internals Program	Rev. 0002
		NPG-SPP-09.26.19	Pressurized Water Reactor Pressure Vessel Integrity	Rev. 0004
		NPG-SPP-09.7	Corrosion Control Program	Rev. 0008
		NPG-SPP-09.7.1	Corrosion Control – General, Localized and Galvanic, and Stress Corrosion Cracking Program	Rev. 0004
		NPG-SPP-09.7.3	Raw Water Corrosion Program	Rev. 0005
		NPG-SPP-09.7.5	Erosion Program	Rev.0000
		NPG-SPP-17.2-	Engineering Training Needs Analysis for Job Specific ESP's,	4/8/19

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		10	ESPO7O.234, ESPO7O.531 and EGT2O2.950	
		NPG-SPP-22.300	Corrective Action Program	Rev. 0017
		NRC Bulletin 88-09	Thimble Tube Thinning in Westinghouse Reactors	7/26/1988
		NUREG-1339	Resolution of Generic Safety Issue 29, Bolting Degradation or Failure in Nuclear Power Plants	June 1990
		PTLR-1	Sequoyah Unit-1, RCS Pressure Temperature Limit Report, Rev. 6, September 2017	4/30/2018
		PTLR-2	Sequoyah Unit 2, Pressure Temperature Limits Report, Revision 7, September 2017	4/30/18
		S10191029803	Diesel Fuel Monitoring Program Commitment Closure Package, Unit 2	08/15/2018
		SQN-1&2-FAC-SNM	SQN-1& 2 Susceptible Non-modeled Analysis for Flow Accelerated Corrosion	2
		SQN-1&2-FAC-SSE	SQN-1&2 system Susceptibility Evaluation for Flow Accelerated Corrosion	1
		SQN-1-ESE	SQN-1 Erosion Susceptibility Evaluation (ESE)	0
		SQN-2-ESE	SQN-2 Erosion Susceptibility Evaluation (ESE)	0
		TBA-TSP-18.802	Requirements for the Safe Operations Cranes	Rev.19
		TI-1	Elastomer and Polymer Aging Management	Rev. 0000
		TI-255	Sequoyah Nuclear Plant, Unit 0, Technical Instruction, Surveillance Program for Reactor Irradiation Specimens	Rev. 0006
		TI-70	Cleanliness of Fluid Systems	Rev. 0037
		Tracking #: 1356623	TVA PM Change/Deferral (Regulatory Impact Example)	7/22/19
		TVA Letter CNL-15-095	Sequoyah Nuclear Plant, Revision to the Reactor Pressure Vessel Surveillance Capsule Withdrawal Schedule for License Renewal	5/14/19
		TVA PM Change/Deferral No.1356623	Inspect and clean After-cooler to Aux Control Air Compressor A-A. Ops Evaluate LCO/ODCM: 3.6.1.8, 3 (Regulatory)	7/22/2019
		TVA065-REPT-	Sample Plan Review, Non-EQ Cable Connections Program	Rev. 3

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		054		
		UFSAR Table 5.2.4-1	Mechanical Property Data for the Closure Head Bolting Material of the Sequoyah Unit 1 and 2 Reactor Vessel	SQN-25
		USNRC Reg, Guide 1.65	Materials and Inspection for Reactor Vessel Closure Studs	Rev.1
	NDE Reports	180-9250039-000	Sequoyah SQN2-R20, Reactor Vessel Internals Visual Examinations	
		EDMS #: L18 180410 800	In-Service Inspection Report No. R-0189, Eddy Current Examination Report, In-core Flux Thimble Tubes, Sequoyah Nuclear Plant Unit 1, Cycle 22	April 2018
		R0066	Record of Reactor Vessel Examination	04/20/2015
	Procedures	0-MI-ECR-079-095	Spent Fuel Bridge Crane Periodic Inspection	Rev. 19
		0-MI-ECR-079-930.0	Manipulator Crane Checkout	Rev. 15
		0-MI-ECR-303-911.0	Reactor Building (Polar) Crane Periodic Inspection	Rev.21
		0-MI-ECR-303-921.0	Auxiliary Building Crane Periodic Inspection	Rev.2
		0-MI-MIN-000-070.0	Cleanliness of Fluid Systems for Maintenance Activities	Rev. 0015
		0-MI-MRR-068-005.0	Removal of Reactor Pressure Vessel Head and Attachments	Rev. 0065
		0-MI-MRR-068-006.0	Installation Of Reactor Pressure Vessel Head And Attachments	Rev. 57
		0-MI-MXX-068-005.0	Steam Generator Primary Side Maintenance Activities	3/22/2019
		0-PI-CEM-000-010.5	Ground Water Monitoring for License Renewal	Rev. 1
		0-PI-CEM-082-001.0	Standby Diesel Generator Starting Air Quality Test	Rev. 0000
		0-PI-DXX-000-100.03.1	Buried and Underground Tanks Inspection	Rev.3
		0-PI-DXX-000-	Periodic Rectifier Monitoring	Rev.0

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		100.03.2		
		0-PI-DXX-000-100.03.3	Periodic InTellusCRM Monitoring	Rev.0
		0-PI-DXX-000-100.03.4	Annual Cathodic Protection Survey	Rev.0
		0-PI-DXX-000-100.07.1	Environmentally Assisted Fatigue Monitoring	Rev. 0000
		0-PI-DXX-000-100.14.1	Internal Surfaces in Miscellaneous Piping and Ducting Components Program Inspections	Rev. 0002
		0-PI-DXX-000-100.21.1	Oil Analysis Inspection, 161 kV Oil-Filled Cables	Rev. 0001
		0-PI-DXX-000-100.31.1	Structures Monitoring for Maintenance Rule and License Renewal	Rev. 2
		0-PI-DXX-000-100.33.1	Water Chemistry Control - Closed Treated Water Inspection	Rev. 0003
		0-PI-DXX-000-105	Boric Acid Leak Monitoring Program	4
		0-PI-DXX-090-000.0	Radiation Monitor Test Data Review	Rev. 0000
		0-PI-ENV-000-002.0	Inspection of Ponds, Channels, Dikes, and Water Control Structures	Rev. 2
		0-PI-MIN-032-001.0	Visual Inspection of the Turbine Building Control Air System	Rev. 0002
		0-PI-MIN-032-002.0	Visual Inspection of Surface Conditions in the Auxiliary Control Air System	Rev. 0001
		0-PI-MIN-082-001.0	Inspection of Surface Conditions in the Diesel Starting Air Subsystem	Rev. 0005
		0-PI-NXX-094-001.0 Rev. 0043	Sequoyah Nuclear Plant, Unit-0, In-Core Seal Table Maintenance	Rev. 0043
		0-PI-SFT-067-003.A	ERCW Train "A", Manipulation for Plant Activities	Rev. 0008
		0-PI-SFT-070-002.0	Performance Testing of Component Cooling Heat Exchangers OB1, OB2	Rev. 0015

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		0-SI-FPU-302-001.R	Fire Barrier Visual Inspection - Auxiliary Building and Control Building	Rev. 0015
		0-SI-FPU-302-004.R	Fire Barrier Visual Inspection - Diesel Generator Building, ERCW Pumping Station and HPFP Pump House	Rev. 0009
		0-SI-FPU-302-005.R	Fire Barrier Visual Inspection - ERCW Manholes and Handholes	Rev. 0009
		0-SI-FPU-302-007.R	Visual Inspection of Electrical Raceway Fire Barrier Systems	Rev. 0017
		0-SI-FPU-410-002.R	Inspection of Reactor Building Airlocks and Blast Doors, Unit 2 Radiant Energy Shield and Unit 1 Accumulator Room 1 Floor	Rev. 0009
		0-SI-MIN-302-239.0	Testing of the Divider Barrier Seal	Rev. 0010
		0-SI-NXX-000-009	Monitoring Component Cyclic or Transient Limits	Rev. 0027
		0-TI-CEM-032-001.0	Turbine Building Control Air Quality Test	Rev. 0002
		0-TI-CEM-032-002.0	Auxiliary Building Control Air Quality Test	Rev. 0002
		0-TI-DXX-000-010.0	Protective Coatings Program	Rev. 0004
		0-TI-SPT-000-301.0	ASME Section XI Pressure Testing Program Basis Document	9
		1-AR-M15-B	Miscellaneous 1-XA-55-15B (Annunciator Response)	Rev. 0045
		1-PI-IXX-092-002.0	Nuclear Instrumentation Power Range Cable Testing	Rev. 0001
		1-SI-FPU-302-001.R	Fire Barrier Visual Inspection - Unit 1 Reactor Building Shield Wall	Rev. 0008
		2-PI-IXX-092-002.0	Nuclear Instrumentation Power Range Cable Testing	Rev. 0001
		2-SI-FPU-302-002.R	Fire Barrier Visual Inspection - Auxiliary Building	Rev. 0003
		54-ISI-603-007	Automated Ultrasonic Examination of RPV Closure Head Penetrations Containing Thermal Sleeves	7

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		54-UT-901-003	Automated Ultrasonic Examination of Wear in RPV Closure Head Thermal Sleeves	3
		DS-M4.2.1	Flow Accelerated Corrosion Program Methods	10
		HSP-209	Physical Properties Measurement Procedure for Aluminum Based Neutron Absorber Surveillance Coupons	Rev. 1
		M&AI-5.3	Application and Repair of Protective Coatings	Rev. 0016
		M&AI-9.1	Bolted Structural Connections (A-307, A-325 and A-490)	8
		M&AI-9.2	Bolted Connections for Unistrut Clamps, Anchor Bolts, Instrument Bolting, and Miscellaneous Bolting	16
		M&AI-9.3	Pipe Clamps for Snubbers/Struts and Associated Bolting	8
		MMTP-104	Guidelines And Methodology For Assembling And Tensioning Threaded Connections	10
		N-VT-1	Visual Examination Procedure for ASME Section XI Preservice and Inservice Non-Destructive Examination	Rev. 0048
		N-VT-22	Visual Examination Procedure for License Renewal Programs	Rev. 0001
		N-VT-8	Remote Visual Examination of PWR Vessel Interiors and Core Support Structures	14
		NEDP-21.0.2	License Renewal Activities Prior to Period of Extended Operation	1
		NEDP-21.0.3	License Renewal Implementation During the Period of Extended Operation (PEO)	1
		NFDP-111	Nuclear Design and Core Analysis, MRP-227 Assumptions, Ref. 6.2 FFF	Rev. 0004
		NPG-SPP-03.3	NRC Commitment Management	Rev. 0007
		NPG-SPP-06.9.2	Surveillance Test Program	9
		NPG-SPP-09.0	Conduct of Engineering	10
		NPG-SPP-09.1	Conduct of System Engineering and Equipment Reliability	6
		NPG-SPP-09.1.1	System Monitoring and Trending	2
		NPG-SPP-09.7	Corrosion Control Program	7
		NPG-SPP-09.7.1	Corrosion Control - General, Localized and Galvanic, and Stress Corrosion Cracking Program	3
		NPG-SPP-09.7.2	Flow Accelerated Corrosion Control Program	4

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		NPG-SPP-09.7.4	Boric Acid Corrosion Control Program	5
		NPG-SPP-09.7.5	Erosion Program	0
		NPG-SPP-22.500	Operating Experience Program	10
		NPG-SPP-6.2	Preventive Maintenance	13
		TVA-NQA-PLN89-A	Nuclear Quality Assurance Plan (NQAP)	37
	Self-Assessments	W50191107003	Review of SQN Plant Documents for the Identification of Newly Identified Systems, Structures, and Components as required by 10 CFR 54.37(b)	11/06/2019
		W51071031001	Review of SQN Plant Documents for the Identification of Newly Identified Systems, Structures, and Components as required by 10 CFR 54.37(b)	08/21/2017
	Work Orders	All Work Orders	114073526, 115364783, 115453612, 116593467, 116880207, 117080857, 117968680, 117977989, 118121028, 118132638, 118193768, 118611647, 118611657, 118611668, 119169548, 119991743, 120158600	
		Preventative Maintenance Tasks (4 digit P-Number)	P1300, P7704, P7772, P7773, P7786, P7907	
		Preventative Maintenance Tasks (5 digit PM Number)	16385, 74433, 74436, 74444, 74447, 78325	
		Preventative Maintenance Tasks (6-digit PM Number)	112883, 120398, 120399, 120446, 120478, 120749, 126922, 126923, 126924	
		Preventative Maintenance Tasks (9 digit PM Number)	014500000, 014510000, 014880001, 014880002, 014880003, 018160000, 030330000, 050201000, 050202000, 058370000, 056860004, 056860005, 056860006, 056860007, 058780001, 058780002, 058780005, 058780006, 058780007, 062950001,	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			063040001, 063601499, 063604819, 063604820, 063604821, 063604822, 063605536, 063605537, 063605538, 063605648, , 800170889, 800170966, 800170967, 800170978, 800170980, 800170981, 800170983, 800170988, 800170989	