

CONTROL BLOCK:

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 (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N C B E P I 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5

7 8 9 14 15 25 26 30 37 CAT 58

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

CON'T

0 1 REPORT SOURCE L 6 0 5 0 - 0 3 2 5 7 1 2 1 7 8 3 8 0 1 1 6 8 4 9

7 8 60 61 68 69 74 75 80

DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

0 2 During unit power operation Remote Shutdown Panel (RSP) reactor vessel pressure indica-

0 3 tor, C32-PI-3332, showed a pressure of 860 psi while redundant Control Room instrumen-

0 4 tation showed an expected pressure of 990 psi. On December 20, 1983, it was determined

0 5 that the following RSP instruments were inoperable: C32-PI-3332, PT-3332; B21-LI-

0 6 3331, LT-3331, LT-N017D-3; LSH-N017D-3; CAC-LI-3342, LT-3342, PI-3341, PT-3341; E11-

0 7 FT-3339, FI-3339, FT-3336, FI-3338, FY-3338; PDT-N002BX; PDI-3334!. Neither event af-

0 8 fected the health and safety of the public.

7 8 9 80

Technical Specifications 3.3.5.2, 6.9.1.9b

0 9 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

7 8 9 10 11 12 13 14 15 16

I C 11 E 12 G 13 I N S T R U 14 P 15 Z 16

17 LER/RO REPORT NUMBER EVENT YEAR 8 3 21 22

23 24 25 26 27 28 29 30 31 32

0 3 0 3 L 0

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS 22 ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

18 19 20 21 22 23 24 25 26

A Z Z Z 0 0 0 0 Y Y L E 2 2 9

33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 0 Both events resulted from a failure of electrical capacitor C2 in the RSP power supply.

1 1 B21-ES-4051. On January 6, 1984, C2, Model No. 164C5261P001 was replaced and ES-4051

1 2 was returned to service. The subject RSP instruments were returned to service. The

1 3 remaining RSP power supplies were checked for proper operation with no problems found.

1 4 No further action is planned regarding these events.

7 8 9 80

FACILITY STATUS % POWER OTHER STATUS 30 METHOD OF DISCOVERY DISCOVERY DESCRIPTION 32

1 5 E 28 0 9 7 29 NA A 31 Operator Surveillance

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36

1 6 Z 33 Z 34 NA NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39

1 7 0 0 0 37 Z 38 NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PERSONNEL INJURIES NUMBER DESCRIPTION 41

1 8 0 0 0 40

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 43

1 9 Z 42 NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PUBLICITY ISSUED DESCRIPTION 45

2 0 N 44 NA

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

NAME OF PREPARER M. J. Pastva, Jr. PHONE 919-457-9521

8401240019 840116 PDR ADOCK 05000325 S PDR

FE22

NRC USE ONLY

78 79 80

LER ATTACHMENT - RO #1-83-60

Facility: Unit No. 1

Event Date: 12-17-83

Operator surveillance during unit power operation on December 17, 1983, revealed reactor pressure indicator 1-C32-PI-3332, located on the Remote Shutdown Panel (RSP), was showing a pressure of 860 psi. At the time, redundant Control Room reactor pressure instrumentation showed an expected indication of 990 psi. On December 20, 1983, initial troubleshooting of the problem affecting PI-3332 revealed that power supply 1-B21-ES-4051 to the RSP had low voltage output with excessive ripple, which rendered the power supply inoperable. It was therefore determined that in addition to PI-3332 and its respective instrument transmitter, PT-3332, the following RSP instruments were inoperable: reactor vessel water level instruments 1-B21-LI-3331 and LT-3331, 1-B21-LT-N017D-3 and LSH-N017D-3; suppression chamber water level instruments 1-CAC-LI-3342 and LT-3342; drywell pressure instruments 1-CAC-PI-3341 and PT-3341; residual heat removal (RHR) head spray flow instruments 1-E11-FT-3338, FI-3338, FY-3338; RHR service water discharge differential pressure instruments 1-E11-PDT-N002BX and PDI-3344.

On January 6, 1984, further investigation of the event revealed the low, erratic output of ES-4051 resulted from an open capacitor, C2, in the power supply circuitry. C2, Elma Engineering Model No. 164C5261P001, was replaced and the ES-4051 was returned to service along with the affected RSP instruments. A check of the remaining RSP instruments showed no apparent problems with their respective power supplies. No further action is planned regarding these events.

CP&L

Carolina Power & Light Company

JAN 19 PM 12:35
Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429
January 16, 1984

FILE: B09-13510C
SERIAL: BSEP/84-0069

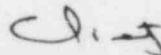
Mr. James P. O'Reilly, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1
DOCKET NO. 50-325
LICENSE NO. DPR-71
LICENSEE EVENT REPORT 1-83-60

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,



C. R. Dietz, General Manager
Brunswick Steam Electric Plant

RMP/sdl/LETJH3

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk

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