



Commonwealth Edison
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December 27, 1983

Mr. James G. Keppler
Regional Administrator
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

Subject: LaSalle County Station Unit 1
High Drywell Temperature
NRC Docket No. 50-373

Reference: (a) C. W. Schroeder letter to J. G. Keppler
dated December 22, 1983 transmitting report
on Unit 1 High Drywell Temperature

Dear Mr. Keppler:

This letter supplements reference (a) to support the immediate restart of LaSalle Unit 1 following the recent overtemperature experience in the drywell. Members of your staff had requested additional information on two topics. The first question concerns the adequacy of the visual inspection technique to determine the condition of cables at the higher elevations of the drywell and specifically what type of verification efforts were used to validate the inspection. The second question addressed the need for assurance that the cable splices incorporated with the replacement cables are capable of in-containment service and conform with industry standards.

As indicated in the referenced report, the visual inspection of cabling in the upper zones of the Unit 1 drywell was correlated with engineering calculations for remaining life based on the same Arrhenius methods used to predict the initial life of the cable. The safety related cables were removed and replaced above drywell elevation 796 feet as were most non-safety related cables. The visually indicated damage was confined to elevations above 804 feet. The analysis indicated that the selection of 796 ft. elevation for cable replacement was valid (see Table IV-1 of the report). To confirm this decision on the replacement elevation, three cables representative of the drywell cabling have been removed for confirmatory testing at Wyle Laboratories. One of these is an SRV actuator cable which had a local hot spot where it was exposed to a heated pipe; the other two cables are vertical run cables that were judged to be undamaged and representative of elevations extending from the lower drywell to the upper drywell. The testing of these cables follows IEEE 383 standards for functionality and acceptance. These cables are also to be tested through a LOCA exposure to validate their "remaining life" by test. Results from the confirmatory tests will be made available to the resident inspector when completed.

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The cable splices were discussed under Section IV-3 on page 8 of the report where it was stated that the splice positions used in the cable replacement were consistent with the original LaSalle criteria, i.e. splices were not used in cable trays, conduits or duct runs but were located inside junction boxes, pull boxes, or at the primary containment penetration connection. These splices were made under approved field installation procedures based upon the vendor's recommendations for cable butt splices. The splices at the penetration position are approved splices; however, because they do not conform to the Commonwealth Edison standard for a single splice (rather than two splices), they will be removed at the first refueling outage and the single conforming splice will be installed at that position.

With these clarifications to the referenced report, Commonwealth Edison's follow-up actions are now on record to assure that the Unit 1 cabling and equipment can credibly function for the "remaining life" represented in the report.

To the best of my knowledge and belief the statements contained herein are true and correct. In some respects these statements are not based on my personal knowledge but upon information furnished by other Commonwealth Edison and contractor employees. Such information has been reviewed in accordance with company practice and I believe it to be reliable.

If there are any further questions in this matter, please contact this office.

Very truly yours,

C. W. Schroeder 12/27/83

C. W. Schroeder
Nuclear Licensing Administrator

GRC/pl

cc: NRC Resident Inspector - LSCS