

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE January 13, 1984
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 948-8551

OPERATING STATUS

		NOTES
1. UNIT NAME:	THREE MILE ISLAND UNIT 1	
2. REPORTING PERIOD:	DECEMBER, 1983.	
3. LICENSED THERMAL POWER (MWT):	2535.	
4. NAMEPLATE RATING (GROSS MWE):	871.	
5. DESIGN ELECTRICAL RATING (NET MWE):	819.	
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE):	824.	
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE):	776.	

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE) _____
10. REASONS FOR RESTRICTIONS, IF ANY: _____

	THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	744.	8760.	81793.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.0	31731.8
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	838.5
14. HOURS GENERATOR ON-LINE	0.0	0.0	31180.9
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.	0.	76531071.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	25484330.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.	0.	23840053.
19. UNIT SERVICE FACTOR	0.0	0.0	38.1
20. UNIT AVAILABILITY FACTOR	0.0	0.0	38.1
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	0.0	37.2
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	0.0	35.6
23. UNIT FORCED OUTAGE RATE	100.0	100.0	58.2

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
 IE24

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE January 13, 1984
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 948-8551

MONTH: DECEMBER

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0.	17	0.
2	0.	18	0.
3	0.	19	0.
4	0.	20	0.
5	0.	21	0.
6	0.	22	0.
7	0.	23	0.
8	0.	24	0.
9	0.	25	0.
10	0.	26	0.
11	0.	27	0.
12	0.	28	0.
13	0.	29	0.
14	0.	30	0.
15	0.	31	0.
16	0.		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December, 1983DOCKET NO. 50-289UNIT NAME TMI-1DATE January 13, 1984COMPLETED BY Courtney SmythTELEPHONE X 8551

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	83/12/01	F	744	D	1	N/A	ZZ	ZZZZZZ	Regulatory Restraint Order

1
F- Forced
S- Scheduled

2 Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3 Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4 Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LIR) File (NURIG-
0161)

5 Exhibit I - Same Source

OPERATIONS SUMMARY

The unit was in cold shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System.

MAJOR SAFETY RELATED MAINTENANCE

The following work was performed during the month:

Snubber Inspection Program:

Work that was generated as a result of an in-depth inspection of all "important to safety" snubbers, was completed last month. In addition to all of the minor repairs made, approximately 20 of the snubbers were totally rebuilt and tested satisfactorily.

Main Stream Safety Relief Valves:

The remainder of the 14 relief valves that had been identified to be weeping during Hot Functional Testing were repaired. This consisted of disassembly, lapping seats and reassembly.

OTSG Work:

The manway and handhole nuts were retorqued on both OTSGs. Two studs that exhibited boron crystallization were removed and inspections were performed to detect both thread wastage and intergranular stress corrosion cracking of the carbon steel studs. No wastage or cracking was observed and the studs were replaced. The remaining studs with lesser deposits were cleaned and lubricated prior to the final torquing.

Decay Heat Cooler IB Inspection:

Because of poor decay heat removal capacity during the last Hot Functional Test, OH-C-1B was inspected (shell side). No fouling was identified. As part of the investigation, the cooler bypass valve (DC-V65B) was found to be partially open when in it's closed position. This valve has been disassembled and repaired. Cooler performance will be watched carefully during the next opportunity. If decay heat cooling capacity continues to be a problem, it will be resolved, however, the repair of the bypass valve is expected to have solved the problem.

Building Spray Pump Repair

BS-P-1B repairs were necessary to replace damaged wear rings that resulted from the pump being run with the suction valve closed. Repairs and testing will be completed in January.

Makeup System Valve Repacking:

A program was conducted to repack Makeup System valves in the Aux. Building. Also repaired were leakoff line tubing leaks and all threaded fittings.

REFUELING INFORMATION SHEET

1. Name of Facility:

Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1988 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).



GPU Nuclear Corporation
Post Office Box 480
Route 441 South
Middletown, Pennsylvania 17057-0191
717 944-7621
TELEX 84-2386
Writer's Direct Dial Number:

January 13, 1984
5211-84-2009

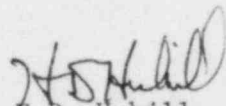
Office of Management Information
and Program Control
Attn: W.C. MacDonald
c/o Distribution Services Branch
DPC, ADM
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
December Monthly Operating Report

Enclosed please find two (2) copies of the December Operating Report for
Three Mile Island Nuclear Station, Unit I.

Sincerely,


H.D. Hukill
Director, TMI-1

HDH:CWS:mle

Enclosures

cc: V. Stello
Dr. Thomas E. Murley

IERA
1/1