

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

DECEMBER 1983

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station Unit One is a Boiling Water Reactor with a designed electrical output of 1078 MWe net, located in Marseilles, Illinois. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. The plant is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit has not commenced commercial generation of power.

This report was compiled by Diane L. Lin, telephone number (815)357-6761, extension 499.

II. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

December 1-31 The unit was shutdown due to a ventilation problem in the drywell and the replacement of cables which were damaged by excessive heat in the drywell.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED
MAINTENANCE

A. Amendments to Facility License or Technical Specifications.

There were no amendments to the facility license or technical specifications during the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

C. Tests and Experiments Requiring NRC Approval.

There were no tests or experiments requiring NRC approval during the reporting period.

D. Corrective Maintenance of Safety Related Equipment.

The following tables present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

ATTACHMENT A
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

LIP-300-7
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WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L26104	---	Damper OVE09YA	Faulty actuator	Damper would not move	Rebuild actuator
L26920 L27753	---	MSIV Leakage Control Main Steam Pressure and Instrument Stop Valves	Stem plating and stem/bonnet threads galled.	Valves broken	Replaced stem and stem/bonnet.
L27531	---	D/G Wattmeter	Defective meter	Meter damaged when D/G 2A was paralleled out of phase with the grid	Replaced meter
L27682	---	DW Air/Supp Pool Water Temperature Recorder	Defective range card	Recorder not driving properly and indicating irrationally	Replaced range card
L28423	83-152/03L-0	RHR Shutdown Isolation Valve	Bottom seat ring sealing surface was not flat	Valve would not pass local leak rate test	Machined bottom seat ring
L29339	83-142/03L-0	RHR Suction Cooling Inboard Isolation Valve	Valve was closed when hot and when cooled stuck	Valve would not open	Opened valve
L29581	83-145/03L-0	Mechanical Snubber	High temperature in drywell	Snubber was inoperable	Replaced snubber
L29744	83-148/03L-0	Switchgear 136X Undervoltage Relay	Contact welded on relay 1327-AP107A	When bus 142Y was de-energ- ized, equipment on 136X did not load shed	Freed up welded contact

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CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

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WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L29889	---	RHR 'C' LPCI Testable Check and bypass Valve Cables	High temperature in drywell	Possible damage to cable insulation	Repulled cable
L29953	---	RCIC Testable Check Valve Cables	High temperature in drywell	Possible damage to cable insulation	Repulled cables
L29971	---	HPCS Testable Check Valve 1E22-F005	High temperature in drywell	Possible damage to cable insulation	Replaced cable
L29978	---	RHR 'B' Heat Exchanger Outlet Valve	Limits out-of-adjustment	To stop valve travel	Adjusted limits
L30120	---	SRV Solenoid Cable	High temperature in drywell	Cable insulation cracked and brittle	Replaced cables
L30292 L30293	---	Mechanical Snubbers	High temperature in drywell	Snubbers inoperable	Replaced snubbers
L30383 L30387 L30389 L30392-6 L30637	---	Mechanical Snubbers	High temperature in drywell	Possible damage to snubbers	Replaced snubbers
L30617	---	SRV Cables	High temperature in drywell	Cables were damaged	Replaced cables
L30879 L30880	---	Cable to SRV	Cable was damaged during pullback	Cable was damaged	Replaced cable

ATTACHMENT A
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

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WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L31110	---	RPV Metal Temperature Recorder	Incorrect thermocouple wiring	Point #4 read downscale and point #2 read upscale	Corrected Thermocouple wiring
L31147	---	'A' RR Pump Breaker 2A	Bad control device	Breaker 2A will not close	Replaced control device

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, December 1 through December 31, 1983. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
83-143/01T-0	11/23/83	Heat Damaged Cables
83-144/03L-0	11/13/83	Unit 1 RHR Service Water Pump 'D' Failure to Start
83-145/03L-0	11/08/83	Inoperable Mechanical Snubber
83-146/03L-0	11/09/83	Containment Leakage Limit Exceeded
83-147/03L-0	11/12/83	Loss of RHR 'B' Shutdown Cooling
83-148/03L-0	11/18/83	Absence of Load Shedding on Switchgear 136X
83-149/03L-0	12/07/83	Rx Vessel Low Water Level Scram / PC Isolation
83-150/01T-0	12/12/83	Incomplete Performance of SRM/IRM Functional Tests
83-151/03L-0	11/28/83	Functional Test of IRM's
83-152/03L-0	11/30/83	Shutdown Cooling Inboard Isolation Valve Leakage (1E12-F009)

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 050-373UNIT LaSalle OneDATE January 6, 1984COMPLETED BY Diane L. LinTELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD: December 1983 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 100% MAX. DEPEND. CAPACITY (MWe-Net): 0
DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): _____
4. REASONS FOR RESTRICTION (IF ANY): _____

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>0</u>	<u>3617.4</u>	<u>6364.8</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0</u>	<u>0</u>	<u>0</u>
7. HOURS GENERATOR ON LINE	<u>0</u>	<u>3086.8</u>	<u>4945.38</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0</u>	<u>0</u>	<u>0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>0</u>	<u>5864648</u>	<u>8005227</u>
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	<u>0</u>	<u>1782781</u>	<u>2304080</u>
11. NET ELECTRICAL ENERGY GENERATED (MWH)	<u>0</u>	<u>1639809</u>	<u>2100584</u>
12. REACTOR SERVICE FACTOR	<u>NA</u>	<u>NA</u>	<u>NA</u>
13. REACTOR AVAILABILITY FACTOR	<u>NA</u>	<u>NA</u>	<u>NA</u>
14. UNIT SERVICE FACTOR	<u>NA</u>	<u>NA</u>	<u>NA</u>
15. UNIT AVAILABILITY FACTOR	<u>NA</u>	<u>NA</u>	<u>NA</u>
16. UNIT CAPACITY FACTOR (Using MDC)	<u>NA</u>	<u>NA</u>	<u>NA</u>
17. UNIT CAPACITY FACTOR (Using Design MWe)	<u>NA</u>	<u>NA</u>	<u>NA</u>
18. UNIT FORCED OUTAGE RATE	<u>NA</u>	<u>NA</u>	<u>NA</u>

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 1-1-84

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	<u>6/21/82</u>
INITIAL ELECTRICITY	_____	<u>9/04/82</u>
COMMERCIAL OPERATION	_____	<u>1/1/84</u>

ATTACHMENT B

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-373UNIT LaSalle OneDATE January 6, 1984COMPLETED BY Diane L. LinTELEPHONE (815) 357-6761MONTH December 1983DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100 line (or the estimated power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

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ATTACHMENT E
UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December 1983

DOCKET NO. 050-373

UNIT NAME LaSalle One

DATE January 6, 84

COMPLETED BY Diane L. Lin

TELEPHONE (815) 357-6761

NO.	DATE	TYPE		DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
42	11/3/83*	S		744.0	B	4	Opened oil circuit breakers 9-10 and 10-11 for STP-27

*Refer to the November 1983 report.

VI. UNIQUE REPORTING REQUIREMENTS

A. Main Steam Relief Valve Operations for Unit 1

Relief valve operations during the reporting period are summarized in the following Table. The table included information as to which relief valve was actuated, how it was activated and the circumstances resulting in its actuation.

<u>Date</u>	<u>Valves Actuated</u>	<u>No & Type Actuations</u>	<u>Plant Conditions</u>	<u>Description of Events</u>
12/2/83	1B21-F013E	10 Manual	0 psig	LIS-NB-14

B. ECCS Systems Outages

The following outages were taken on ECCS System during the reporting period:

<u>Outage No.</u>	<u>Equipment</u>	<u>Purpose of Outage</u>
1-1395-83	'B' RHR Heat Exchanger Outlet Valve	Hydro 1E12-F003B
1-1396-83	'B' and 'C' RHR Water Leg Pump 1E12-C003	Lube Coupling

C. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculations Manual during this reporting period.

D. Radioactive Waste Treatment System

There were no changes to the Radioactive Waste Treatment System during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

January 5, 1984

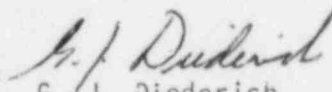
Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station, Unit One, for the period covering December 1 through December 31, 1983.

Very truly yours,


G. J. Diederich
Station Superintendent
LaSalle County Station

GJD/DLL/bej

Enclosure

xc: J. G. Keppler NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright III, Dept. of Nuclear Safety
D. P. Galle CECO
D. L. Farrar CECO
INPO Records Center
Ron A. Johnson, P/P Coordinator SNED

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