TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
400 Chestnut Street Tower II

January 9, 1984

Director of Nuclear Reactor Regulation
Attention: Ms. E. Adensam, Chief
Licensing Branch No. 4
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Ms. Adensam:

In the Matter of	)	Docket No. 50-327
Tennessee Valley Authority	)	50-328

By my November 23, 1983 letter to you, we requested NRC approval of a proposed license amendment for the Sequoyah Nuclear Plant, units 1 and 2. The proposed license amendment would allow completion of the modification associated with Postaccident Sampling (PAS) in order to comply with the operating license conditions, 2.C.(23).F for unit 1 and 2.C.(16).g for unit 2 and the requirements of Item II.B.3 of NUREG-0737. The details of the design and compliance with the design criteria of NUREG-0737 were provided in the November 23, 1983 letter. The justification for approval of the proposed license amendment, along with the significant hazards consideration determination pursuant to 10 CFR 50.92, was provided by my December 21, 1983 letter to you.

Enclosed is a revised response to criterion 11.B that was previously submitted to you by the November 23, 1983 letter.

If you have any questions concerning this matter, please get in touch with Jerry Wills at FTS 858-2683.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

L. M. Mills, Manager
Nuclear Licensing

Sworn to and subscribed before me
this 9th day of Jan 1984

Bryant M. Fowers
Notary Public
My Commission Expires 4/8/86

Enclosure
cc: U.S. Nuclear Regulatory Commission (Enclosure)
Region II
Attn: Mr. James P. O'Reilly Administrator
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30303

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ENCLOSURE

REVISED RESPONSE TO CRITERION 11.B
POSTACCIDENT SAMPLING NUREG-0737, ITEM II.B.3
SEQUOYAH NUCLEAR PLANT

Criterion

11.B The ventilation exhaust from the sampling station should be filtered with charcoal adsorbers and high-efficiency particulate air (HEPA) filters.

Response

During normal plant operation, ventilation air is supplied to the PASF via the auxiliary building general ventilation system and an auxiliary supply fan. Exhaust air is ducted directly to the Auxiliary Building general ventilation system.

During postaccident conditions or sampling operations, the normal supply and exhaust systems are isolated, and ventilation air is taken directly from the outside at a point on the roof of the unit 1 additional equipment building. Both the unit 1 and unit 2 systems share this common intake. A supply fan provides air to the sampling side of the facility in response to a differential pressure controller. Air is drawn from both the sample and valve gallery areas by an exhaust fan. This air then passes through the postaccident sampling facility gas treatment system (PASF GTS) air cleanup unit and is then routed to the exhaust duct downstream of the Auxiliary Building gas treatment system (ABGTS) air cleanup unit. The sampling area is maintained at a positive pressure of 0.125 inch water gauge (WG) with respect to atmosphere while the valve gallery is kept at a negative pressure of -0.25 inch WG with respect to the sample side.

The radiological gas treatment subsystem, of the PASF GTS, consists of one HEPA/Charcoal-type air cleanup unit located just upstream of the exhaust fan. Air supplied to the facility during postaccident conditions or sampling operations is processed through the air cleanup unit before being discharged to the atmosphere.

The postaccident sampling facility environmental control system is not a nuclear safety-related system. However, it has redundant isolation capability in all ductwork that interfaces with the ABGTS or penetrates the auxiliary building secondary containment enclosure (ABSCE). The isolation valves and ductwork which interface with the ABGTS and ABSCE are designed to seismic category 1 criteria. Also, the isolation valves are backed by Class 1E power. All remaining portions of the system are designed to seismic category I(L) criteria requirements.