



BOSTON EDISON

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

R. A. Anderson
Vice President &
Station Director
Nuclear Operations

July 11, 1991
BECO Ltr. #9*-088

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

License No. DPR-35
Docket No. 50-293

Subject: June 1991 Monthly Report

Dear Sir:

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.


R.A. Anderson

WJM/bal

Attachment

cc: Regional Administrator, Region 1
U.S. Nuclear Regulatory Commission
475 Allendale Rd.
King of Prussia, PA 19406

Senior Resident Inspector

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
 UNIT Pilgrim I
 DATE July 11, 1991
 COMPLETED BY W. Munro
 TELEPHONE (508) 747-8474

MONTH June 1991

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>	17	<u>0</u>
2	<u>0</u>	18	<u>0</u>
3	<u>0</u>	19	<u>0</u>
4	<u>0</u>	20	<u>0</u>
5	<u>0</u>	21	<u>0</u>
6	<u>0</u>	22	<u>0</u>
7	<u>0</u>	23	<u>0</u>
8	<u>0</u>	24	<u>0</u>
9	<u>0</u>	25	<u>0</u>
10	<u>0</u>	26	<u>0</u>
11	<u>0</u>	27	<u>0</u>
12	<u>0</u>	28	<u>0</u>
13	<u>0</u>	29	<u>0</u>
14	<u>0</u>	30	<u>0</u>
15	<u>0</u>	31	<u>N/A</u>
16	<u>0</u>		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE July 11, 1991
 COMPLETED BY W. Munro
 TELEPHONE (508) 747-8474

OPERATING STATUS

Notes

1. Unit Name Pilgrim 1
2. Reporting Period June 1991
3. Licensed Thermal Power (Mwt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted, If Any (Net MWe) None

10. Reasons For Restrictions, If Any N/A

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	<u>720.0</u>	<u>4343.0</u>	<u>162671.0</u>
12. Number Of Hours Reactor Was Critical	<u>0.0</u>	<u>2850.2</u>	<u>95450.9</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>2850.2</u>	<u>91766.3</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>0.0</u>	<u>5276592.0</u>	<u>158618088.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>0.0</u>	<u>1812180.0</u>	<u>53459694.0</u>
18. Net Electrical Energy Generated (MWH)	<u>0.0</u>	<u>1743822.0</u>	<u>51370230.0</u>
19. Unit Service Factor	<u>0.0</u>	<u>65.6</u>	<u>56.4</u>
20. Unit Availability Factor	<u>0.0</u>	<u>65.6</u>	<u>56.4</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>59.9</u>	<u>47.1</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>61.3</u>	<u>48.2</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>3.6</u>	<u>12.5</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling Outage No. 8, approximately 85 days			

25. If Shut Down At End Of Report Period, Estimated Date of Startup 7/23/91

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

Operational Summary for June 1991

Continued refueling outage number 8.

Safety Relief Valve Challenges
Month of June 1991

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during this reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 dated 01/05/81.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that, each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next Refueling Shutdown: April 3, 1993
3. Scheduled date for restart following present refueling: July 23, 1991
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1991 refueling outage was of the same design as loaded in the previous outage, and consisted of 168 assemblies.
7. (a) There are 429 fuel assemblies in the core.
(b) There are 1640* fuel assemblies in the spent fuel pool.
8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
(b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 831 fuel assemblies.

* Includes 151 fuel assemblies in the spent fuel pool which will be returned to the core for Cycle 9. Also includes one (1) reconstituted fuel bundle in the spent fuel pool containing 1 failed fuel rod discovered during sipping procedure.

Month June 1991

PILGRIM NUCLEAR POWER STATION
MAJOR SAFETY RELATED MAINTENANCE

<u>SYSTEM</u>	<u>COMPONENT</u>	<u>MALFUNCTION</u>	<u>CAUSE</u>	<u>MAINTENANCE</u>	<u>CORRECTIVE ACTION TO PREVENT RECURRENCE</u>	<u>ASSOCIATED LER</u>
Salt Service Water System (SSW)	Loop "A" piping within the intake structure.	Identified wall thinning.	Excessive corrosion and delamination of rubber lining within spool pieces.	Removed and replaced various rubber lined spool pieces within the intake structure Temporary Modification TM 91-32 installed replacing two rubber lined spool pieces with two unlined spool pieces.	Unlined spool pieces will be replaced during mid-cycle outage (1992).	N/A
Reactor Building Closed Cooling Water System (RBCCW)	Heat Exchanger E-209 "A".	Cracks on heat exchanger partition plates.	Metal fatigue	Installation of six (6) stiffener plates to each partition plate per FRN 91-03-20, providing greater resistance to fatigue failure.	Perform inspection during RFO-9.	N/A
		Tubes leaking	Corrosion, and inlet end erosion.	Thirteen (13) tubes required plugging and twenty nine (29) other tubes required replugging.		
Feed-water System (FW)	Check Valves 6 CK-58A and B and 6 CK-62 A and B	Failed LLRT surveillance procedure.	Hinge pin misalignment causing disks to seat incorrectly.	Replaced disks, installed new hinge pin configuration and replaced soft seat material.	Refer to associated LER.	LER 90-013-00 (To be issued)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293

NAME Pilgrim 1DATE July 11, 1991COMPLETED BY W. MunroTELEPHONE (508) 747-8474REPORT MONTH June 1991

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR ³	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
8 (continuation)	05/04/91	S	720.0	C	4	N/A	N/A	N/A	Refueling outage No. 8

1	2	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Maint or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)