

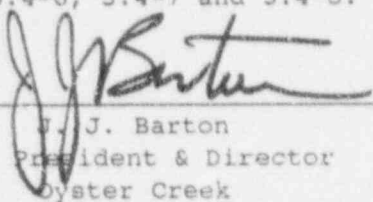
GPU NUCLEAR CORPORATION
OYSTER CREEK NUCLEAR GENERATING STATION

PROVISIONAL OPERATING
LICENSE NO. DPR-16

Technical Specification
Change Request No. 160 Rev. 2
Docket No. 50-219

Applicant submits, by this Technical Specification Change Request No. 160, Rev. 2 to the Oyster Creek Nuclear Generating Station Technical Specifications, a change to pages 3.4-1, 3.4-5, 3.4-6, 3.4-7 and 3.4-8.

By: _____


J. J. Barton
Vice President & Director
Oyster Creek

Sworn and subscribed to before me this 18th day of June 1991.


NOTARY PUBLIC OF NEW JERSEY

JUDITH M. CROWE
Notary Public of New Jersey
My Commission Expires 11/25/95

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

IN THE MATTER OF)
)
GPU NUCLEAR CORPORATION)

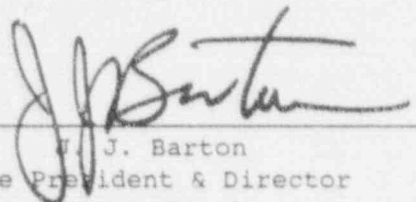
DOCKET NO. 50-219

CERTIFICATE OF SERVICE

This is to certify that a copy of Technical Specification Change Request No. 160, Rev. 2 for the Oyster Creek Nuclear Generating Station Technical Specifications, filed with the United States Nuclear Regulatory Commission on June 18 , 1991, has this day of June 18 , 1991, been served on the Mayor of Lacey Township, Ocean County, New Jersey by deposit in the United States mail, addressed as follows:

The Honorable Debra Madensky
Mayor of Lacey Township
818 West Lacey Road
Forked River, NJ 08731

By: _____


W. J. Barton
Vice President & Director
Oyster Creek

DATED:

OYSTER CREEK NUCLEAR GENERATING STATION
PROVISIONAL OPERATING LICENSE NO. DPR-16
DOCKET NO. 50-219
TECHNICAL SPECIFICATION CHANGE REQUEST NO. 160, REV. 2

Applicant hereby requests the Commission to change Appendix A to the above captioned license as indicated below. Pursuant to 10CFR50.91, an analysis concerning the determination of no significant hazards considerations is also presented:

1. Section to be Changed

3.4 "Emergency Cooling"

2. Extent of Change

Section 3.4 changed to incorporate the 10CFR50.46 LOCA analysis that is the basis for the MAPLHGR limits provided in Technical Specification Section 3.10 "Core Limits." The Limiting Conditions for Operation (LCOs) and Bases have been changed as appropriate.

3. Changes Requested

As indicated on the attached revised Technical Specification pages 3.4-1, 3.4-5, 3.4-6, 3.4-7 and 3.4-8.

4. Discussion

GPUN has recently held discussions with the NRC staff concerning the requirements of 10CFR50.46 and the Technical Specification LCOs for the Core Spray System (CSS). The staff stated that during operation with one core spray loop inoperable, which is a currently existing LCO, the remaining operable loop must meet the requirements of 10CFR50.46, Appendix K. The staff's position was that the inoperable core spray loop constitutes a single failure which must be addressed when demonstrating compliance with 10CFR50.46, Appendix K. TSCR No. 160, Rev. 2 incorporates the 10CFR50.46, Appendix K analysis into the Bases of Section 3.4 and changes the CSS LCOs such that the staff's position is reflected. The LCOs have been changed such that the APLHGR limits are dependent upon the level of CSS availability.

NEDC-31462P "OYSTER CREEK NUCLEAR GENERATION STATION SAFER/CORECOOL/GESTRA-LOCA LOSS-OF-COOLANT ACCIDENT ANALYSIS," August 1987; provides the results of LOCA analysis performed in accordance with NRC requirements and demonstrates conformance with ECCS acceptance criteria of 10CFR50.46. NEDC-31462P has been reviewed and approved by the staff via License Amendment #129 concerning the Cycle 12 reload application. The analysis demonstrates that the 10CFR50.46 criteria will be met assuming a coincident flow of 3400 gpm from one CSS sparger and 2200 gpm from the other CSS sparger. The Oyster Creek CSS consists of two loops; each containing a core spray sparger, two main pumps and two booster pumps. Note that the CSS loop associated with the cracked sparger would be required to supply 3640 or 2360 gpm, as appropriate.

The existing Specification 3.4.A.1 requires CSS component availability which is consistent with that assumed in the NEDC-31462P analysis and therefore need not be changed.

GPUN Topical Report 053 "Thermal Limits With One Core Spray Sparger" demonstrates (consistent with NEDE-30010A, "Performance Evaluation of the Oyster Creek Core Spray Sparger," January 1984 and the NRC's associated Safety Evaluation dated July 20, 1984) that with an APLHGR of less than or equal to 90% of the limits provided in Specification 3.10.A, the acceptance criteria of 10CFR50.46 will be met assuming 3400 gpm from a single sparger at 110 psig.

Proposed Specification 3.4.A.3 provides the action statements required when a CSS loop or its core spray header delta P instrumentation becomes inoperable during the run mode. TSCR No. 160, Rev. 2 modifies the action that the remaining loop contain no inoperable components and be demonstrated daily to be operable, the APLHGR limits will be brought to 90% of the limits given in Specification 3.10.A within two hours of the loop being determined inoperable. With these actions the CSS will provide flow to the core sufficient enough to meet the acceptance criteria of 10CFR50.46.

Proposed Specification 3.4.A.4 provides the action statements required when one of the redundant active loop components in the CSS becomes inoperable during the run mode. TSCR No. 160, Rev. 2 modifies the action statement such that it distinguishes between an inoperable core spray booster pump and an inoperable core spray main pump. With respect to a core spray booster pump, the action statement remains the same, that is, the other core spray booster pump in the loop must be demonstrated daily to be operable. This action is appropriate based upon the LOCA analysis represented by NEDC-31462P. With respect to a core spray main pump, the action statement is changed such that, in addition to retaining the existing requirement to demonstrate that the other core spray main pump in the loop is operable daily, the APLHGR limits will be brought to 90% of the limits given in Specification 3.10.A within two hours of the component being determined inoperable. This action is appropriate based upon the GPUN Topical Report 053.

5. Determination

GPU Nuclear has determined that operation of the Oyster Creek Nuclear Generating Station in accordance with the proposed technical specifications does not involve a significant hazard. The changes do not:

1. Involve a significant increase in the probability or the consequence of an accident previously evaluated. The probability of an accident is not dependent upon APLHGR limits. There are no changes to plant configuration, availability of safety systems, the manner in which the safety systems are initiated or the way the plant is operated that will increase the probability or consequences of an accident.
2. Create the possibility of a new or different kind of accident from any previously evaluated. The proposed change does not alter the plant configuration, nor does it change the availability of safety systems or the manner in which they respond to initiating events. As such, the possibility of a new or different kind of accident from any previously evaluated is not created.
3. Involve a significant reduction in a margin of safety. The proposed APLHGR limits are based upon analysis results which were performed in accordance with methods and procedures approved by NRC for use at Oyster Creek, thus the margin of safety is not reduced.