



PEACH BOTTOM—THE POWER OF EXCELLENCE

**PHILADELPHIA ELECTRIC COMPANY**

PEACH BOTTOM ATOMIC POWER STATION

R. D. 1, Box 208  
Delta, Pennsylvania 17314

(717) 456-7014

June 14, 1991

Docket No. 50-278

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

SUBJECT: Licensee Event Report  
Peach Bottom Atomic Power Station - Unit 3

This LER concerns an Engineered Safety Feature actuation during Surveillance testing due to personnel error.

Reference:	Docket No. 50-278
Report Number:	3-91-008
Revision Number:	00
Event Date:	05/20/91
Report Date:	06/14/91
Facility:	Peach Bottom Atomic Power Station RD 1, Box 208, Delta, PA 17314

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Sincerely,

A handwritten signature in dark ink, appearing to read "John J. Lyash".

cc: J. J. Lyash, USNRC Senior Resident Inspector  
T. T. Martin, USNRC, Region I

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## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Peach Bottom Atomic Power Station - Unit 3										DOCKET NUMBER (2) 0 5 0 0 0 2 7 8 1 OF 0 3										PAGE (3) 1 OF 0 3																													
TITLE (4) Engineered Safety Feature Actuation During Surveillance Testing Due to Personnel Error																																																	
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																			
MONTH			DAY			YEAR				YEAR			SEQUENCE NUMBER			REVISION NUMBER			MONTH			DAY			YEAR				FACILITY NAMES										DOCKET NUMBERS										
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OPERATING MODE (9) N										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (check one or more of the following) (11)																																							
POWER LEVEL (10) 0 0 0										20.402(a) 20.405(a)(1)(i) 20.405(a)(7)(i) 20.405(a)(11)(i) 20.405(a)(1)(iv) 20.405(a)(1)(v)										20.405(c) 50.36(c)(1) 50.36(c)(2) 50.73(a)(2)(i) 50.73(a)(2)(ii) 50.73(a)(2)(iii) 50.73(a)(2)(iv)										X 50.73(a)(2)(iv) 50.73(a)(2)(iv) 50.73(a)(2)(iv) 50.73(a)(2)(iv)(A) 50.73(a)(2)(iv)(B) 50.73(a)(2)(iv)										73.73(a) 73.73(a) OTHER (Specify in Appendix Drawings and in Text, NRC Form 366A)									
LICENSEE CONTACT FOR THIS LER (12)																																																	
NAME A. A. Fulvio, Regulatory Engineer																				TELEPHONE NUMBER AREA CODE 7 1 7 4 5 6 - 7 0 1 4																													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																	
CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC			CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC																						
SUPPLEMENTAL REPORT EXPECTED (14)																																																	
YES (if yes, complete EXPECTED SUBMISSION DATE)																				X NO										EXPECTED SUBMISSION DATE (15)																			
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On 5/20/91 at 0730 hours during the performance of a Residual Heat Removal "B" Channel Logic System Functional Surveillance Test (ST), the 3C RHR pump unexpectedly started when a test switch was inadvertently left in the wrong position during testing. The system was secured and the ST was completed satisfactorily. The cause has been attributed to personnel error caused by a repetitive pattern set up in the ST which was then suddenly changed. The event has been discussed with the involved individuals and the pertinent information has been presented to other I&C personnel. Additionally, the ST will be enhanced. No safety consequences occurred as a result of this event. There was one other previous, similar event that involved the mispositioning of a test switch.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)  Peach Bottom Atomic Power Station Unit 3	DOCKET NUMBER (2)  0 5 0 0 0 2 7 8 9 1	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
			0 0 8	0 0	0 2	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Requirement for the Report

This report is being submitted to satisfy the requirements of 10 CFR 50.73(a)(2)(iv) due to an automatic actuation of an Engineered Safety Feature.

Unit Status at Time of the Event

Unit 3 was in the REFUEL mode. There were no other systems, structures, or components that were inoperable that contributed to the event.

Description of the Event

On 5/20/91 at 0730 hours during the performance of a Residual Heat Removal (RHR)(EIIS:BO) "B" Logic System Functional Surveillance Test (ST), the 3C RHR pump (EIIS:P) unexpectedly started when a test switch (EIIS:HS) was inadvertently left in the wrong position during testing. The purpose of the test switch is to allow logic testing without a pump initiation. The test switch was then returned to the proper position. Prompt notification was made to the NRC via the ENS at 0910 hours. Subsequently, the RHR ST was completed satisfactorily.

Cause of the Event

The cause of this event has been attributed to personnel error (inability, non-licensed) caused by a repetitive pattern set up in the test which was then suddenly changed. During the ST, a key is inserted into an ECCS test switch, the switch is then moved to the test position, an annunciator is verified, the switch is returned to normal, and the annunciator is cleared. This sequence is repeated on the first three switches. However, in the next sequence the fourth switch was also returned to normal rather than being left in the test position as specified in the ST.

Analysis of the Event

No safety consequences occurred as a result of this event. The actuated system operated properly and was restored after the event.

Corrective Actions

The system was secured and the ST was completed satisfactorily.

The event has been discussed with the involved individuals with appropriate actions taken. The pertinent information from this event has been presented to other I&C personnel.

Additionally, the procedure will be enhanced to make the sequence of steps the same for all four switches.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/88

FACILITY NAME (1)  Peach Bottom Atomic Power Station Unit 3	DOCKET NUMBER (2)  0 5 0 0 0 2 7 8	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Previous Similar Events

There was one previous similar event (LER-3-90-13) that involved mispositioning of a test switch. This previous problem was caused by a Maintenance technician misunderstanding the use of a temporary test switch. The corrective action taken as a result of the previous LER involved changing responsibility for ST performance from the Maintenance group to the I&C group. Since this event was the result of a human factors problem, the corrective actions taken as part of the previous LER would not have prevented this event.