



**BOSTON EDISON**  
Pilgrim Nuclear Power Station  
Rocky Hill Road  
Plymouth, Massachusetts 02360

**R. A. Anderson**  
Vice President &  
Station Director  
Nuclear Operations

June 14, 1991  
BECO Ltr. #91- 80

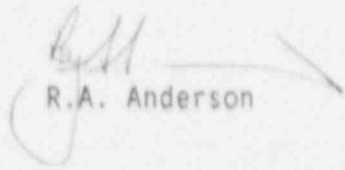
U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

License No. DPR-35  
Docket No. 50-293

Subject: May 1991 Monthly Report

Dear Sir:

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

  
R.A. Anderson

WJM/bal

Attachment

cc: Regional Administrator, Region 1  
U.S. Nuclear Regulatory Commission  
475 Allendale Rd.  
King of Prussia, PA 19406

Senior Resident Inspector

9106180457 910531  
PDR ADOCK 05000293  
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# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293  
 UNIT Pilgrim I  
 DATE June 14, 1991  
 COMPLETED BY W. Munro  
 TELEPHONE (508) 747-8474

MONTH May 1991

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>	17	<u>0</u>
2	<u>0</u>	18	<u>0</u>
3	<u>0</u>	19	<u>0</u>
4	<u>0</u>	20	<u>0</u>
5	<u>0</u>	21	<u>0</u>
6	<u>0</u>	22	<u>0</u>
7	<u>0</u>	23	<u>0</u>
8	<u>0</u>	24	<u>0</u>
9	<u>0</u>	25	<u>0</u>
10	<u>0</u>	26	<u>0</u>
11	<u>0</u>	27	<u>0</u>
12	<u>0</u>	28	<u>0</u>
13	<u>0</u>	29	<u>0</u>
14	<u>0</u>	30	<u>0</u>
15	<u>0</u>	31	<u>0</u>
16	<u>0</u>		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

# OPERATING DATA REPORT

DOCKET NO. 50-293  
 DATE June 14, 1991  
 COMPLETED BY W. Munro  
 TELEPHONE (508) 747-8474

## OPERATING STATUS

### Notes

1. Unit Name Pilgrim 1
2. Reporting Period May 1991
3. Licensed Thermal Power (Mwt) 1998
4. Nameplate Rating (Gross MWe) 678
5. Design Electrical Rating (Net MWe) 655
6. Maximum Dependable Capacity (Gross MWe) 696
7. Maximum Dependable Capacity (Net MWe) 670
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted, If Any (Net MWe) None
10. Reasons For Restrictions, If Any N/A

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours In Reporting Period	<u>744.0</u>	<u>3623.0</u>	<u>161951.0</u>
12. Number Of Hours Reactor Was Critical	<u>0.0</u>	<u>2850.2</u>	<u>95450.9</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>2850.2</u>	<u>91766.3</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>0.0</u>	<u>5276592.0</u>	<u>158618088.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>0.0</u>	<u>1812180.0</u>	<u>53459694.0</u>
18. Net Electrical Energy Generated (MWH)	<u>0.0</u>	<u>1743822.0</u>	<u>51370230.0</u>
19. Unit Service Factor	<u>0.0</u>	<u>78.7</u>	<u>56.7</u>
20. Unit Availability Factor	<u>0.0</u>	<u>78.7</u>	<u>56.7</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>71.8</u>	<u>47.3</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>73.5</u>	<u>48.4</u>
23. Unit Forced Outage Rate	<u>100.0</u>	<u>3.6</u>	<u>12.5</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			
Refueling Outage No. 8, approximately 70 days			

25. If Shut Down At End Of Report Period, Estimated Date of Startup 7/13/91

BOSTON EDISON COMPANY  
PILGRIM NUCLEAR POWER STATION  
DOCKET NO. 50-293

Operational Summary for May 1991

The unit started the reporting period in cold shutdown. Refueling outage Number 8 commenced at 0700 hours on May 4, 1991.

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Safety Relief Valve Challenges  
Month of May 1991

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during this reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Ref. BECo ltr. #81-01 dated 01/05/81.

### REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that, each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next Refueling Shutdown: On or about April 1, 1993
3. Scheduled date for restart following present refueling: Third Quarter 1991
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1986/87 refueling outage was of the same design as loaded in the previous outage, and consisted of 192 assemblies.
7. (a) There are no fuel assemblies in the core.  
(b) There are 2068\* fuel assemblies in the spent fuel pool.
8. (a) The station is presently licensed to store 2320 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.  
(b) The planned spent fuel storage capacity is 2320 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 832 fuel assemblies.

\* Includes 168 new fuel assemblies to support current refuel outage.  
580 fuel assemblies in the spent fuel pool will be returned to the core for Cycle 9.

PILGRIM NUCLEAR POWER STATION  
MAJOR SAFETY RELATED MAINTENANCE

<u>SYSTEM</u>	<u>COMPONENT</u>	<u>MALFUNCTION</u>	<u>CAUSE</u>	<u>MAINTENANCE</u>	<u>CORRECTIVE ACTION TO PREVENT RECURRENCE</u>	<u>ASSOCIATED LER</u>
Recirculation System (RECIRC)	Loop "B" Pump P201B	Increase in Drywell floor sump leakage rate to greater than 5 gpm (F&MR 91-122)	End of useful life of the inner seal of Recirculation pump "B".	Seals to be replaced in both A and B Recirculation System pumps during RFO-8.	Frequency of seal replacement under review. Refer to associated LER.	LER 91-007-00
Primary Containment Isolation Control System (PCIS)	N/A	False high reactor water signal caused three automatic PCIS Group 1 Isolations. (F&MRs 91-124, 91-125 and 91-127)	Root Cause investigation ongoing.	Following the first two isolations, the isolation signal was reset and the main steam line (MSL) drain valve and the main steam isolation (MSIV) valves were reopened, to facilitate decay heat removal. Following the third isolation, the isolation signal was reset and the MSL drain valve and the MSIVs were left closed, using the Residual Heat Removal System for plant cooldown.	To be determined.	LER 91-008-00

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-293

NAME Pilgrim 1DATE June 14, 1991COMPLETED BY W. MunroTELEPHONE (508) 747-8474REPORT MONTH May 1991

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTTING DOWN REACTOR <sup>3</sup>	LICENSE EVENT REPORT #	SYSTEM CODE <sup>4</sup>	COMPONENT CODE <sup>5</sup>	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
8 (continuation)	05/01/91	F	79.0	D	4	91-007-00	AD	P Seal	Seal failure on "B" Recirculation Pump. Seal to be replaced.
8 (continuation)	05/04/91	S	665.0	C	4	N/A	N/A	N/A	Refueling outage No. 8

1	2	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Maint or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination	F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)