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**DUKE POWER**

December 20, 1994

Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: McGuire Nuclear Station  
Docket Nos: 50-369 and 370  
Generic Letter 89-10  
Proposed Schedule Extension

Dear Sir:

As stated in Duke Power Company letter of October 27, 1994, attached is information required by Generic Letter 89-10, Supplement 6 to support proposed schedule extension of the McGuire GL 89-10 program. The proposed completion date for the Group 1 valves is extended to July 19, 1995 and the proposed completion schedule for the Group 2 valves has been accelerated from December 29, 1998 to July 19, 1995.

The attached materials provide appropriate details required by GL 89-10, Supplement 6 including the proposed schedule adjustments, background information, current program status, schedule extension justification, and technical information for each unverified valve.

Should you require further information, please contact James E. Snyder at (704) 875-4447.

Very truly yours,

A handwritten signature in dark ink, appearing to read 'T. C. McMeekin'.  
T. C. McMeekin

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US Nuclear Regulatory Commission  
12-20-94

Page 2/2

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U. S. Nuclear Regulatory Commission  
December 20, 1994

T. C. McMeekin, being duly sworn, states that he is Vice President of Duke Power Company, McGuire Nuclear Site; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this revision to the McGuire Nuclear Station License numbers NPF-9 and NPF-17, and that all the statements and matters set forth therein are true and correct to the best of his knowledge.

T. C. McMeekin  
T. C. McMeekin  
McGuire Nuclear Station

Subscribed and sworn to before me this 20<sup>th</sup> day of December.

Kathy S. Morales  
Notary Public

My Commission Expires:

12/13/98

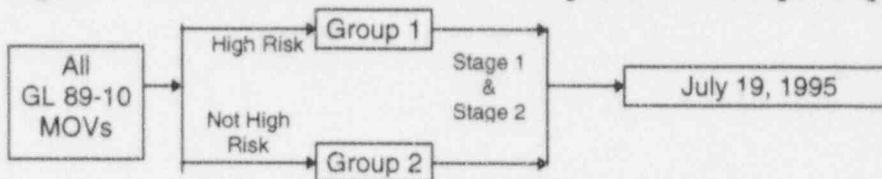
**McGuire Nuclear Station  
NRC Generic Letter 89-10  
Request for Schedule Adjustment**

## **Proposed Schedule Adjustments**

In accordance with the guidelines established by Generic Letter 89-10, Supplement 6, McGuire is proposing two adjustments to the present schedule:

- For Group 1 and low-margin Group 2 MOVs, it is requested that the scheduled completion date for certain Stage 2 activities be extended to July 19, 1995.
- For the balance of Group 2 MOVs, it is proposed that the scheduled completion date for all Stage 1 and Stage 2 activities be adjusted to July 19, 1995. This constitutes an improvement of 3.5 years over the previous schedule.

**Figure 1 - MNS GL 89-10 Schedule - Proposed Revision per Supplement 6**



## **Background**

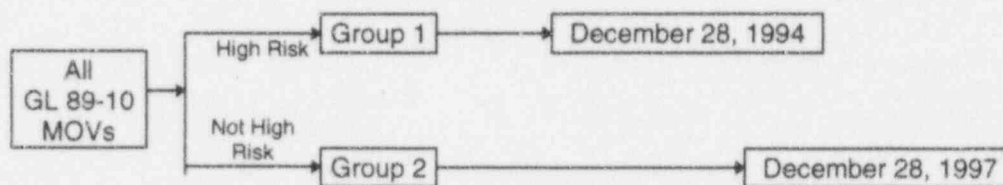
NRC justification for issuing Generic Letter 89-10 was increased theoretical core melt frequency resulting from higher-than-originally-expected MOV failure rates. NUREG/CR-5140, "Value-Impact Analysis for Extension of NRC Bulletin 85-03 to Cover All Safety-Related MOVs," documents this justification based on reported Bulletin 85-03 failure rates, cost estimates from MOVATS Incorporated, a "best estimate" MOV population of 100 per unit (low of 80, high of 130), conducting full design-basis differential pressure testing on 10 percent of the applicable population, and replacing 1 percent due to "uncorrectable deficiencies."

Very early in the program, Duke Power Company (Duke) recognized that their unit average MOV population for McGuire Nuclear Station (MNS) far exceeded the estimated "high" of 130. In the Duke initial response to GL 89-10 (dated December 28, 1989), a risk-based schedule was specified based on Phase 1 (high risk) and Phase 2 (not high risk) MOVs. Later, this terminology was changed to Group 1 (high risk) and Group 2 (not high risk) MOVs. Group 1 and Group 2 are defined below:

- **Group 1 MOVs** are those that are active and are important to core melt scenarios. They are the most safety significant MOVs and are given the highest priority.
- **Group 2 MOVs** consist of the balance of 89-10 MOVs not in Group 1. They are less safety significant than Group 1 MOVs.

Completion schedule for Group 1 was specified as 5 years from December 28, 1989 (December 28, 1994) or 3 refueling outages after June 28, 1990, whichever is longer. For Group 2, this schedule was specified as 8 years from December 28, 1989 (December 28, 1997) or 6 refueling outages after June 28, 1990, whichever is longer.

**Figure 2 - MNS GL 89-10 Initial Schedule**



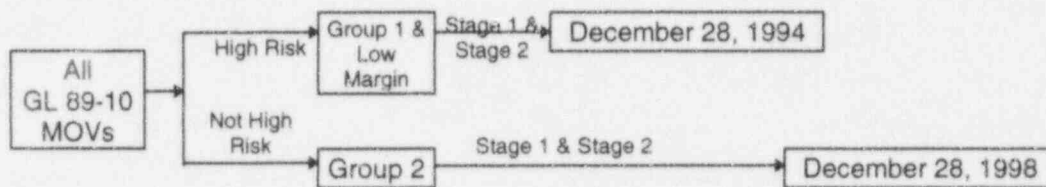
On February 5, 1991, Duke personnel met with the NRC staff to discuss various aspects of the GL 89-10 program. As a result of this meeting, the NRC staff requested that Duke submit a detailed schedule to provide information on the completion of Stage 2. This request was documented in a NRC letter dated March 18, 1991. In a letter dated October 31, 1991, Duke provided the following definitions for Stage 1 and Stage 2:

- **Stage 1** - initial base-line diagnostic testing to verify valve function-ability based on "best available" information.
- **Stage 2** - performing flow and differential pressure testing on a population of MOVs in combination with engineering analyses to validate the initial Stage 1 set-up methodology.

In addition, this letter also specified the following schedule:

- Stage 1 and Stage 2 programs for **Group 1 and marginal Group 2** MOVs will be completed by December 28, 1994 or within 3 refueling outages starting after June 28, 1990, whichever is longer.
- Stage 1 and Stage 2 programs for the **remaining Group 2** MOVs will be completed by December 28, 1998 or within 6 refueling outages starting after June 28, 1990, whichever is longer.

**Figure 3 - MNS GL 89-10 Schedule - October, 1991 Revision**



This schedule is documented on page 13 of the Oconee Phase 1 inspection report dated August 8, 1991. In regards to the schedule, this report states:

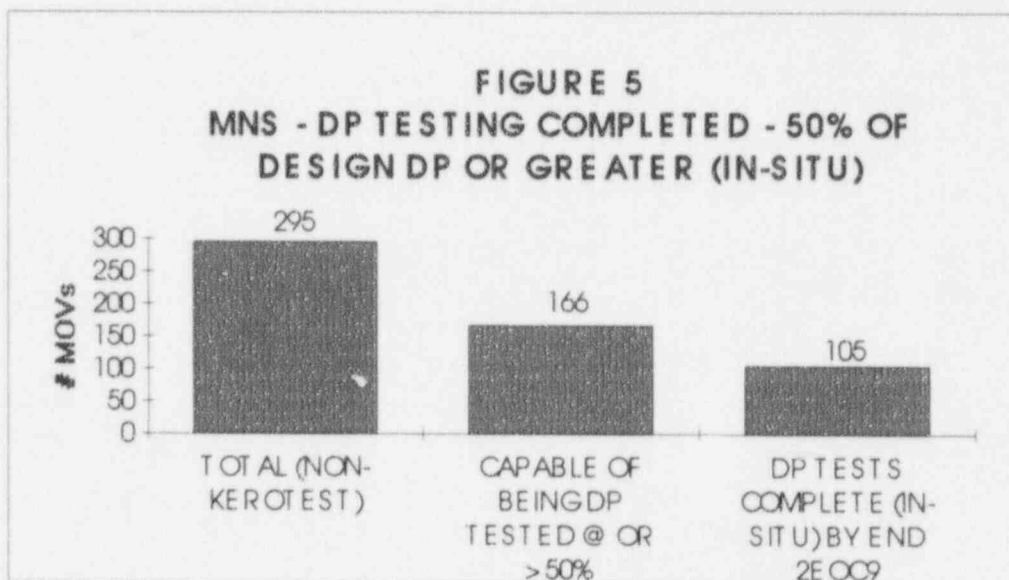
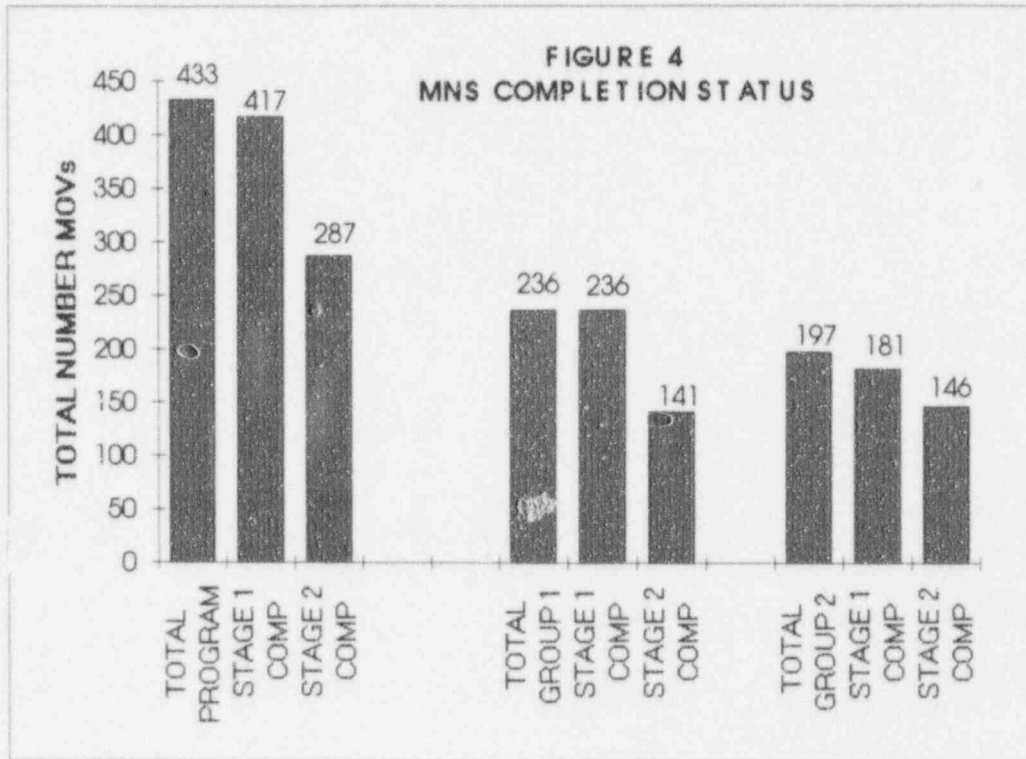
This is a concern, but due to the licensee's efforts to test the highest priority MOVs first, the large number of MOVs that require testing, and the small population of MOVs that will be tested beyond the schedule recommendations, the licensee's schedule for completion is acceptable. The licensee should continue to evaluate its schedule during the testing program to ensure that any necessary revisions are made.

McGuire Phase 1 inspection report dated May 12, 1992 also document this completion schedule.

## Current Program Status

Figure 4 provides the McGuire Nuclear Station (MNS) GL 89-10 program completion status for refueling outages beginning on or before December 28, 1994 and ending January 22, 1995:

Figure 5 shows the level of completion of DP testing projected by refueling outages beginning on or before December 28, 1994 and ending January 22, 1995:





### **Mechanical Improvements Achieved or In Progress**

McGuire has performed a number of modifications to improve the performance of MOVs. The table below shows the approximate number of modification activities performed. Attachment 2 provides a detailed summary of the modifications performed at each of the three sites. Note that there are no outstanding corrective actions based on GL 89-10 program to date.

MODIFICATION TYPE	NUMBER OF MODIFICATIONS PERFORMED	INCLUDED IN ATTACHMENT 2
REPLACE OPERATOR ONLY	24	YES
REPLACE SPRING PACK ONLY	8	YES
TORQUE SWITCH WIRING BYPASSES (completed for all gate and butterfly valves)	293	NO

\*Modifications are only counted once, by the most significant activity.

### **Technical Improvements in Progress**

Duke has initiated several test programs which will be completed at approximately the same time as the original schedule completion date for the Group 1 valves. Duke is also participating in industry-sponsored programs intended to improve the understanding of MOVs. These programs have been adjusted and modified in response to emerging regulatory and industry issues and, in their final form, are expected to be useful for the completion of Stage 2 of the Duke GL 89-10 program. These activities include:

#### **KEROTEST FLOW LOOP TESTING (WATER)**

23 Kerotest globe valves representative of the installed population at the Duke plants have been tested with water under controlled conditions at differential pressures ranging from 0 psid to over 2,500 psid and flow velocities in excess of 200 feet per second. The results of these tests are applicable to approximately 300 MOVs in the Duke program.

#### **KEROTEST VALVE TESTING (STEAM)**

3 Kerotest globe valves representative of the installed population at the Duke plants are planned to be tested with steam under controlled conditions at differential pressures of approximately 2,500 psid and flow velocities up to blowdown conditions. These tests are applicable to Kerotest globe valves in steam service.

#### **EPRI MOV PERFORMANCE PREDICTION PROGRAM**

Duke has actively supported the EPRI MOV Program and anticipates utilizing this methodology in selected cases, pending industry review and final resolution of comments.



### **Program Completion**

Since the original schedule was established, Stage 1 and Stage 2 test activities have been implemented for both Group 1 and Group 2 valves. Emphasis, however, has been placed on the Group 1 valves in recognition of their greater safety significance.

Several techniques, singularly or in combination, will be used for Stage 2 justification of the remaining MOVs. Listed below are the techniques anticipated to be used:

1. **FULL DP TESTING.** Design basis testing of valve. This is the application of test results taken at or near design basis (accident) conditions for the specific valve in question.
2. **PARTIAL DP TESTING.** This is the application of test results taken at less than design basis (accident) conditions for the specific valve in question.
3. Duke in-house test data or outside Duke test data (**GROUPING**). This is the application of test data from other similar or identical valves that are tested at a Duke nuclear plant.
4. **PROTOTYPE TESTING.** This is the application of test results taken for a prototype valve or valves that are tested at a flow loop facility.
5. EPRI MOV Program (**EPRI PPP**) results. This is the application of EPRI flow loop and in-situ test results and/or the actual validated program methodologies.
6. Siemens similarity analysis (**SIMILARITY**). This is the application of similarity results similar to that being performed at Grand Gulf.
7. **MARGIN ANALYSIS** coupled with other technique(s) analysis. This is the process of demonstrating adequate switch set-up margin based on extremely conservative assumptions and use of test results of similar valves.

In accordance with GL 89-10 Supplement 6, Attachment 1 contains a detailed list of each valve within the scope of this extension request, along with the available sizing factor (valve factor for gate and globe valves) for each MOV. The expected Stage 2 justification method is also listed for each MOV.

### **Extension Justification**

McGuire Nuclear Station believes that a schedule extension to July 19, 1995 for certain Stage 2 Group 1 activities is justified because:

- 1) The large number of MOVs in McGuire's 89-10 program (433 as compared to a high estimate of 260, 130 per unit). This has resulted in a large amount of required testing and with several changes being identified in the 89-10 program (6 supplements to the G.L.), this testing is being completed in the current outage (2EOC9) which started within the 89-10 time frame. With testing just completing, some time is needed to factor that data into verification bases. Therefore, the extension of 6 months is requested for Group 1 MOVs.

- 2) McGuire Group 2 MOVs will be completed well ahead of the previous schedule -- an improvement of approximately 3.5 years.
- 3) All McGuire GL 89-10 MOVs have design bases identified and all but 16 Group 2 MOVs have been set-up using conservative sizing factors. The best available sizing factor data have been employed in the existing setups and the functionality of MOVs is confirmed by actually performing diagnostic testing.
- 4) These improvements are expected to exceed the potential impact of an extension to July 19, 1995.

### **Attachments**

The following attachments are provided for McGuire GL 89-10 program MOVs for which Stage 2 will not be completed in refueling outages beginning before December 28, 1994 and ending January 22, 1995:

<u>Attachment</u>	<u>Description</u>
1	McGuire MOVs Applicable to Schedule Extension Request
2	McGuire Modifications
3	Table of Abbreviations

**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

SEQUENCE No.	VALVE NO	VALVE SIZE & TYPE	DESIGN DP PSID	DESIGN FLOW GPM	SAFETY GROUP (1/2)	SAFETY DESCRIPTION	AVAIL. SIZING FACTOR	STATIC TEST DATE	PLANNED STAGE 2 JUSTIFICATION METHODS	PLANNED STAGE 2 COMPLETION DATE
1	0RN0002	36-IN. BF	22	28200	2	RC CROSSOVER SUPPLY TO RN A TRAIN.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
2	0RN0003	36-IN. BF	22	28219	2	RC CROSSOVER SUPPLY TO RN A TRAIN.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
3	0RN0004	36-IN. BF	22	28220	2	RC CROSSOVER SUPPLY TO RN B TRAIN.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
4	0RN0005	36-IN. BF	22	28219	2	RC CROSSOVER SUPPLY TO RN B TRAIN.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
5	0RN0007	36-IN. BF	16	25470	1	STANDBY NUC S/W POND SUPPLY TO RN A TRAIN.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
6	0RN0009	36-IN. BF	16	25470	1	STANDBY NUC S/W POND SUPPLY TO RN B TRAIN.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
7	0RN0010	36-IN. BF	22	28220	1	LOW LEVEL INTAKE SUPPLY TO RN B TRAIN.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
8	0RN0011	36-IN. BF	22	28220	1	LOW LEVEL INTAKE SUPPLY TO RN B TRAIN.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
9	0RN0012	36-IN. BF	25	28220	1	LOW LEVEL INTAKE SUPPLY TO RN A TRAIN.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
10	0RN0013	36-IN. BF	25	28219	1	LOW LEVEL INTAKE SUPPLY TO RN A TRAIN.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
11	0RN0014	36-IN. BF	24	28219	2	RN SUPPLY TRAIN CROSSOVER.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
12	0RN0015	36-IN. BF	24	28219	2	RN SUPPLY TRAIN CROSSOVER.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
13	0RN0147	36-IN. BF	22	28220	1	RN A TRAIN DISCHARGE HEADER TO RC	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
14	0RN0148	36-IN. BF	29	28220	1	RN A TRAIN DISCHARGE HEADER TO RC	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
15	0RN0149	36-IN. BF	21	25500	2	RN A TRAIN SNSWP DISCHARGE.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
16	0RN0150	36-IN. BF	19	14500	2	RN A TRAIN DISCHARGE HEADER CROSSOVER.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
17	0RN0151	36-IN. BF	22	14500	2	RN B TRAIN DISCHARGE HEADER CROSSOVER.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95

**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

SEQUENCE No.	VALVE NO	VALVE SIZE & TYPE	DESIGN DP PSID	DESIGN FLOW GPM	SAFETY GROUP (1/2)	SAFETY DESCRIPTION	AVAIL. SIZING FACTOR	STATIC TEST DATE	PLANNED STAGE 2 JUSTIFICATION METHODS	PLANNED STAGE 2 COMPLETION DATE
18	0RN0152	36-IN. BF	20	25500	1	RN B TRAIN SNSWP DISCHARGE.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
19	0RN0283	36-IN. BF	26	28220	1	RN A TRAIN DISCHARGE TO RC HEADER.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
20	0RN0284	36-IN. BF	29	28220	1	RN B TRAIN DISCHARGE TO RC HEADER.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
21	0RN0301	24-IN. BF	23	2673	2	RN SUPPLY TO RV SYSTEM.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
22	0RN0302	24-IN. BF	23	2673	2	RN SUPPLY TO RV SYSTEM.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
23	1CA0007	8-IN GT	117	N/A	1	THE NON-SAFETY-RELATED CONDENSATE SOURCE	0.5	11/8/91	EPRI PPP, GROUPING	2nd QRT 95
24	1CA0009	8-IN GT	117	N/A	1	THE NON-SAFETY-RELATED CONDENSATE SOURCE	0.5	11/5/91	EPRI PPP, GROUPING	2nd QRT 95
25	1CA0011	8-IN GT	117	N/A	1	FROM NON-SAFETY-RELATED CONDENSATE SOURCE	0.5	11/9/91	EPRI PPP, GROUPING	2nd QRT 95
26	1CA0015	6-IN GT	116	N/A	1	PROVIDES TRAIN RELATED FLOWPATH FOR THE CA MDP A FROM SAFETY RELATED ASSURED RN SUPPLY. OPEN FUNCTION.	0.35	11/29/91	EPRI PPP,MARGIN ANALYSIS, GROUPING	2nd QRT 95
27	1CA0018	6-IN GT	116	N/A	1	PROVIDES TRAIN RELATED FLOWPATH FOR THE CA MDP B FROM SAFETY RELATED ASSURED RN SUPPLY. OPEN FUNCTION.	0.35	10/18/91	EPRI PPP,MARGIN ANALYSIS, GROUPING	2nd QRT 95
28	1CA0050	4-IN GT	2075	N/A	1	PROVIDES ISOLATION FOR S/G C FROM THE TDP IN THE EVENT OF A FAULTED S/G.	0.5	12/5/91	EPRI PPP, GROUPING	2nd QRT 95
29	1CA0066	4-IN GT	2075	N/A	1	PROVIDES ISOLATION FOR S/G A FROM THE TDP IN THE EVENT OF A FAULTED S/G.	0.5	10/14/91	EPRI PPP, GROUPING	2nd QRT 95
30	1CA0086	8-IN GT	116	N/A	1	PROVIDES TRAIN RELATED FLOWPATH FOR CA TDP FROM SAFETY RELATED ASSURED RN SUPPLY. OPEN FUNCTION.	0.35	5/31/92	EPRI PPP,MARGIN ANALYSIS, GROUPING	2nd QRT 95

**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

SEQUENCE No.	VALVE NO	VALVE SIZE & TYPE	DESIGN DP PSID	DESIGN FLOW GPM	SAFETY GROUP (1/2)	SAFETY DESCRIPTION	AVAIL. SIZING FACTOR	STATIC TEST DATE	PLANNED STAGE 2 JUSTIFICATION METHODS	PLANNED STAGE 2 COMPLETION DATE
31	1CA0116	8-IN GT	116	N/A	1	PROVIDES TRAIN RELATED FLOWPATH FOR CA TDP FROM SAFETY RELATED ASSURED RN SUPPLY. OPEN FUNCTION.	0.35	5/22/92	EPRI PPP,MARGIN ANALYSIS, GROUPING	2nd QRT 95
32	1CA0161	8-IN GT	50	N/A	1	PROVIDES THE FLOWPATH FOR THE CA TO PUMP FROM THE RC SYSTEM IN A SSS EVENT. OPEN FUNCTION	0.35	5/21/93	EPRI PPP,MARGIN ANALYSIS, GROUPING	2nd QRT 95
33	1CA0162	8-IN GT	50	N/A	1	PROVIDES THE FLOWPATH FOR THE CA TO PUMP FROM THE RC SYSTEM IN A SSS EVENT. OPEN FUNCTION	0.35	5/26/93	EPRI PPP,MARGIN ANALYSIS, GROUPING	2nd QRT 95
34	1CF0126	6-IN GT	862	N/A	2	SG PREHEATER BYPASS ISOLATION TO THE AUXILIARY FEEDWATER NOZZLES.	0.5	5/30/89	EPRI PPP, GROUPING	2nd QRT 95
35	1CF0127	6-IN GT	862	N/A	2	SG PREHEATER BYPASS ISOLATION TO THE AUXILIARY FEEDWATER NOZZLES.	0.5	5/29/89	EPRI PPP, GROUPING	2nd QRT 95
36	1CF0128	6-IN GT	862	N/A	2	SG PREHEATER BYPASS ISOLATION TO THE AUXILIARY FEEDWATER NOZZLES.	0.5	7/25/90	EPRI PPP, GROUPING	2nd QRT 95
37	1CF0129	6-IN GT	862	N/A	2	SG PREHEATER BYPASS ISOLATION TO THE AUXILIARY FEEDWATER NOZZLES.	0.5	5/30/89	EPRI PPP, GROUPING	2nd QRT 95
38	1LD0108	4-IN GT	57	N/A	2	BYPASS THE FUEL LUBE OIL FILTER ON HIGH DIFFERENTIAL PRESSURE. OPEN FUNCTION.	0.35	2/3/93	OPEN, LARGE MARGIN	2nd QRT 95
39	1LD0113	4-IN GT	57	N/A	2	BYPASS THE FUEL LUBE OIL FILTER ON HIGH DIFFERENTIAL PRESSURE. OPEN FUNCTION.	0.35	3/3/93	OPEN, LARGE MARGIN	2nd QRT 95
40	1NC0031	3-IN GT	2335	N/A	1	ISOLATE THE INLET TO EACH NCS PORV	0.5	5/31/93	EPRI PPP, GROUPING	2nd QRT 95
41	1NC0033	3-IN GT	2335	N/A	1	ISOLATE THE INLET TO EACH NCS PORV	0.5	5/27/93	EPRI PPP, GROUPING	2nd QRT 95
42	1NC0035	3-IN GT	2335	N/A	1	ISOLATE THE INLET TO EACH NCS PORV	0.5	5/28/93	EPRI PPP, GROUPING	2nd QRT 95



**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

SEQUENCE No.	VALVE NO	VALVE SIZE & TYPE	DESIGN DP PSID	DESIGN FLOW GPM	SAFETY GROUP (1/2)	SAFETY DESCRIPTION	AVAIL. SIZING FACTOR	STATIC TEST DATE	PLANNED STAGE 2 JUSTIFICATION METHODS	PLANNED STAGE 2 COMPLETION DATE
43	1NC0056	4-IN GT	159	N/A	2	CONTAINMENT ISOLATION FROM THE REACTOR MAKEUP WATER PUMPS AND HEADER TO THE PRT.	0.5	2/6/94	EPRI PPP, GROUPING, SIMILARITY	2nd QRT 95
44	1ND0001	14-IN GT	386	N/A	1	ISOLATE THE NC SYSTEM FROM THE ND SYSTEM.	0.5	4/29/93	EPRI PPP, GROUPING	2nd QRT 95
45	1ND0002	14-IN GT	530	N/A	1	ISOLATE THE NC SYSTEM FROM THE ND SYSTEM. PROVIDE CONTAINMENT ISOLATION DURING A SSS EVENT.	0.5	10/30/91	EPRI PPP, GROUPING	2nd QRT 95
46	1ND0004	14-IN GT	200	N/A	1	PROVIDE ND PUMP SUCTION ISOLATION DURING THE RECIRCULATION PHASE OF AN ECCS ACTUATION.	0.5	4/21/93	EPRI PPP, GROUPING	2nd QRT 95
47	1ND0019	14-IN GT	200	N/A	1	PROVIDE ND PUMP SUCTION ISOLATION DURING THE RECIRCULATION PHASE OF AN ECCS ACTUATION.	0.5	4/28/93	EPRI PPP, GROUPING	2nd QRT 95
48	1NI0009	4-IN GT	2710	N/A	1	ALLOW HIGH PRESSURE SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM (FROM THE NV PUMPS).	0.5	10/16/91	EPRI PPP, GROUPING	2nd QRT 95
49	1NI0010	4-IN GT	2710	N/A	1	ALLOW HIGH PRESSURE SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM (FROM THE NV PUMPS).	0.5	10/16/91	EPRI PPP, GROUPING	2nd QRT 95
50	1NI0054	10-IN GT	633	N/A	1	ALLOW COLD LEG ACCUMULATOR SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM	0.5	5/4/93	EPRI PPP, GROUPING	2nd QRT 95
51	1NI0065	10-IN GT	633	N/A	1	ALLOW COLD LEG ACCUMULATOR SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM	0.5	5/20/93	EPRI PPP, GROUPING	2nd QRT 95
52	1NI0076	10-IN GT	633	N/A	1	ALLOW COLD LEG ACCUMULATOR SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM	0.5	5/19/93	EPRI PPP, GROUPING	2nd QRT 95
53	1NI0088	10-IN GT	633	N/A	1	ALLOW COLD LEG ACCUMULATOR SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM	0.5	9/2/93	EPRI PPP, GROUPING	2nd QRT 95

**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

SEQUENCE No.	VALVE NO	VALVE SIZE & TYPE	DESIGN DP PSID	DESIGN FLOW GPM	SAFETY GROUP (1/2)	SAFETY DESCRIPTION	AVAIL. SIZING FACTOR	STATIC TEST DATE	PLANNED STAGE 2 JUSTIFICATION METHODS	PLANNED STAGE 2 COMPLETION DATE
54	1NI0100	8-IN GT	200	N/A	1	OPEN DURING THE COLD LEG INJECTION PHASE OF ECCS OPERATION TO ALLOW FLOW FROM THE RWST TO THE SIPS.	0.35	4/23/93	GROUPING, SIMILARITY	2nd QRT 95
55	1NI0121	4-IN GT	1004	N/A	2	OPEN TO ALLOW S/P DISCHARGE INTO THE RCS HOT LEG DURING THE HOT LEG RECIRCULATION PHASE OF ECCS OPERATION.	0.5	11/30/91	GROUPING, SIMILARITY	2nd QRT 95
56	1NI0152	4-IN GT	1004	N/A	2	OPENS TO ALLOW SIP DISCHARGE INTO THE RCS HOT LEG DURING THE HOT LEG RECIRCULATION PHASE OF ECCS OPERATION.	0.5	10/11/91	GROUPING, SIMILARITY	2nd QRT 95
57	1NS0001	12-IN GT	68	N/A	1	OPENED TO REALIGN THE SPRAY PUMPS SUCTION FROM THE RWST TO THE CONTAINMENT SUMP.	0.5	5/29/93	GROUPING, SIMILARITY	2nd QRT 95
58	1NS0003	12-IN GT	69	N/A	1	CLOSED TO REALIGN THE SPRAY PUMPS SUCTION FROM THE RWST TO THE CONTAINMENT SUMP.	0.5	5/24/93	GROUPING, SIMILARITY	2nd QRT 95
59	1NS0012	8-IN GT	63	N/A	1	OPEN UPON RECEIPT OF A SP SIGNAL AND STAY OPEN TO PROVIDE FLOW TO THE SPRAY HEADERS.	0.5	3/29/93	EPRI PPP, GROUPING	2nd QRT 95
60	1NS0015	8-IN GT	63	N/A	1	OPEN UPON RECEIPT OF A SP SIGNAL AND STAY OPEN TO PROVIDE FLOW TO THE SPRAY HEADERS.	0.5	3/30/93	EPRI PPP, GROUPING	2nd QRT 95
61	1NS0018	12-IN GT	68	N/A	1	OPENED TO REALIGN THE SPRAY PUMPS SUCTION FROM THE RWST TO THE CONTAINMENT SUMP.	0.5	5/21/93	GROUPING, SIMILARITY	2nd QRT 95
62	1NS0020	12-IN GT	69	N/A	1	CLOSED TO REALIGN THE SPRAY PUMPS SUCTION FROM THE RWST TO THE CONTAINMENT SUMP.	0.5	5/23/93	GROUPING, SIMILARITY	2nd QRT 95
63	1NS0029	8-IN GT	63	N/A	1	OPEN UPON RECEIPT OF A SP SIGNAL AND STAY OPEN TO PROVIDE FLOW TO THE SPRAY HEADERS.	0.5	4/14/93	EPRI PPP, GROUPING	2nd QRT 95
64	1NS0032	8-IN GT	63	N/A	1	OPEN UPON RECEIPT OF A SP SIGNAL AND STAY OPEN TO PROVIDE FLOW TO THE SPRAY HEADERS.	0.5	4/1/93	EPRI PPP, GROUPING	2nd QRT 95



**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

SEQUENCE No.	VALVE NO	VALVE SIZE & TYPE	DESIGN DP PSID	DESIGN FLOW GPM	SAFETY GROUP (1/2)	SAFETY DESCRIPTION	AVAIL. SIZING FACTOR	STATIC TEST DATE	PLANNED STAGE 2 JUSTIFICATION METHODS	PLANNED STAGE 2 COMPLETION DATE
65	1NS0038	8-IN GT	204	N/A	1	OPENED TO PROVIDE A FLOWPATH FROM THE RHR PUMPS TO THE AUXILIARY CONTAINMENT SPRAY HEADERS.	0.5	4/16/93	EPRI PPP, GROUPING	2nd QRT 95
66	1NS0043	8-IN GT	204	N/A	1	OPENED TO PROVIDE A FLOWPATH FROM THE RHR PUMPS TO THE AUXILIARY CONTAINMENT SPRAY HEADERS.	0.5	4/16/93	EPRI PPP, GROUPING	2nd QRT 95
67	1NV0094	4-IN GT	134	N/A	2	CONTAINMENT ISOLATION FOR THE REACTOR COOLANT PUMP SEAL RETURN.	0.5	10/25/88	GROUPING, SIMILARITY	2nd QRT 95
68	1NV0095	4-IN GT	150	N/A	2	CONTAINMENT ISOLATION FOR THE REACTOR COOLANT PUMP SEAL RETURN.	0.5	11/13/91	GROUPING, SIMILARITY	2nd QRT 95
69	1NV0141	4-IN GT	89	N/A	2	ISOLATION FOR THE VOLUME CONTROL TANK AND THE SUCTION OF THE RECIPROCATING CHARGING PUMPS.	0.5	10/8/91	GROUPING, SIMILARITY	2nd QRT 95
70	1NV0142	4-IN GT	89	N/A	2	ISOLATION FOR THE VOLUME CONTROL TANK AND THE SUCTION OF THE RECIPROCATING CHARGING PUMPS.	0.5	10/7/91	GROUPING, SIMILARITY	2nd QRT 95
71	1NV0221	8-IN GT	183	N/A	1	PROVIDE A FLOWPATH FROM THE RWST TO THE NV PUMPS. CLOSE TO ISOLATE THE RWST FROM THE NV PUMPS.	0.5	5/12/93	GROUPING, SIMILARITY	2nd QRT 95
72	1NV0222	8-IN GT	183	N/A	1	PROVIDE A FLOWPATH FROM THE RWST TO THE NV PUMPS. CLOSE TO ISOLATE THE RWST FROM THE NV PUMPS.	0.5	9/1/93	GROUPING, SIMILARITY	2nd QRT 95
73	1RN0016	36-IN. BF	8	14500	1	ISOLATION OF RN TRAIN SUPPLY TO RN PUMP.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
74	1RN0018	36-IN. BF	8	14500	1	ISOLATION OF RN TRAIN SUPPLY TO RN PUMP.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
75	1RN0042	10-IN. BF	121	98	2	RN SUPPLY TO THE AUX BLDG NON-ESSENTIAL HEADER.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
76	1RN0279	6-IN. BF	4	750	2	UNIT 1 AUX BLDG VENT UNITS DISCHARGE TO RN SYSTEM.	NA	1EOC9 (9/94)	MARGIN ANALYSIS, GROUPING	2nd QRT 95

**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

SEQUENCE No.	VALVE NO	VALVE SIZE & TYPE	DESIGN DP PSID	DESIGN FLOW GPM	SAFETY GROUP (1/2)	SAFETY DESCRIPTION	AVAIL SIZING FACTOR	STATIC TEST DATE	PLANNED STAGE 2 JUSTIFICATION METHODS	PLANNED STAGE 2 COMPLETION DATE
77	1RN0299	6-IN. BF	3	750	2	UNIT 1 RV AUX BLDG VENT UNITS DISCHARGE TO RN SYSTEM.	NA	1EOC9 (9/94)	MARGIN ANALYSIS, GROUPING	2nd QRT 95
78	1RV0032	12-IN. BF	106	4080	2	CONT. ISOL. FOR THE LOWER CONTAINMENT VENT UNITS AND THE INCORE INSTRUMENT ROOM VENT UNITS.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
79	1RV0033	12-IN. BF	106	4080	2	CONT. ISOL. FOR THE LOWER CONTAINMENT VENT UNITS AND THE INCORE INSTRUMENT ROOM VENT UNITS.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
80	1RV0076	12-IN. BF	106	4080	2	CONT. ISOL. FOR THE LOWER CONTAINMENT VENT UNITS AND THE INCORE INSTRUMENT ROOM VENT UNITS.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
81	1RV0077	12-IN. BF	106	4080	2	VENT UNITS AND THE INCORE INSTRUMENT ROOM VENT UNITS.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
82	1WL0321	6-IN. BF	91	0	1	CONTAINMENT ISOLATION FOR THE VUCDT INPUTS.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
83	1WL0322	6-IN. BF	91	0	1	CONTAINMENT ISOLATION FOR THE VUCDT INPUTS.	NA	1EOC9 (9/94)	EPRI PPP, GROUPING	2nd QRT 95
84	2CA0007	8-IN GT	117	N/A	1	THE NON-SAFETY-RELATED CONDENSATE SOURCE	0.5	2/12/92	EPRI PPP, GROUPING	2nd QRT 95
85	2CA0009	8-IN GT	117	N/A	1	THE NON-SAFETY-RELATED CONDENSATE SOURCE	0.5	2/14/92	EPRI PPP, GROUPING	2nd QRT 95
86	2CA0011	8-IN GT	117	N/A	1	ISOLATES THE ASSURED RN SOURCE FROM THE NON-SAFETY-RELATED CONDENSATE SOURCE	0.5	2/13/92	EPRI PPP, GROUPING	2nd QRT 95
87	2CA0015	6-IN GT	116	N/A	1	PROVIDES THE TRAIN RELATED FLOWPATH FOR THE CA MDP A FROM THE SAFETY RELATED ASSURED RN SUPPLY FOR A DBA. OPEN FUNCTION	0.35	11/10/95	EPRI PPP, GROUPING	2nd QRT 95
88	2CA0018	6-IN GT	116	N/A	1	PROVIDES THE TRAIN RELATED FLOWPATH FOR THE CA MDP B FROM THE SAFETY RELATED ASSURED RN SUPPLY FOR A DBA. OPEN FUNCTION	0.35	11/17/95	EPRI PPP, GROUPING	2nd QRT 95

**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

SEQUENCE No.	VALVE NO	VALVE SIZE & TYPE	DESIGN DP PSID	DESIGN FLOW GPM	SAFETY GROUP (1/2)	SAFETY DESCRIPTION	AVAIL. SIZING FACTOR	STATIC TEST DATE	PLANNED STAGE 2 JUSTIFICATION METHODS	PLANNED STAGE 2 COMPLETION DATE
89	2CA0038	4-IN GT	2075	N/A	1	ISOLATION FOR S/G D FROM THE TDP IN THE EVENT OF A FAULTED S/G.	0.5	2/27/92	EPRI PPP, GROUPING	2nd QRT 95
90	2CA0050	4-IN GT	2075	N/A	1	ISOLATION FOR S/G C FROM THE TDP IN THE EVENT OF A FAULTED S/G.	0.5	1/22/92	EPRI PPP, GROUPING	2nd QRT 95
91	2CA0054	4-IN GT	2075	N/A	1	ISOLATION FOR S/G B FROM THE TDP IN THE EVENT OF A FAULTED S/G.	0.5	8/25/93	EPRI PPP, GROUPING	2nd QRT 95
92	2CA0058	4-IN GT	1605	N/A	1	ISOLATION FOR S/G B FROM THE MDP A IN THE EVENT OF A FAULTED S/G.	0.5	2/29/92	EPRI PPP, GROUPING	2nd QRT 95
93	2CA0066	4-IN GT	2075	N/A	1	ISOLATION FOR S/G A FROM THE TDP IN THE EVENT OF A FAULTED S/G.	0.5	1/27/92	EPRI PPP, GROUPING	2nd QRT 95
94	2CA0086	8-IN GT	116	N/A	1	PROVIDES THE TRAIN RELATED FLOWPATH FOR THE CA TDP FROM THE SAFETY RELATED ASSURED RN SUPPLY FOR A DBA. OPEN FUNCTION.	0.35	7/1/93	EPRI PPP, GROUPING, MARGIN ANALYSIS	2nd QRT 95
95	2CA0116	8-IN GT	116	N/A	1	PROVIDES THE TRAIN RELATED FLOWPATH FOR THE CA TDP FROM THE SAFETY RELATED ASSURED RN SUPPLY FOR A DBA. OPEN FUNCTION.	0.35	6/29/93	EPRI PPP, GROUPING, MARGIN ANALYSIS	2nd QRT 95
96	2CA0161	8-IN GT	116	N/A	1	PROVIDES THE FLOWPATH FOR THE CA TDP TO PUMP FROM THE RC SYSTEM IN A SSS EVENT. OPEN FUNCTION.	0.35	7/12/93	EPRI PPP, MARGIN ANALYSIS, GROUPING	2nd QRT 95
97	2CA0162	8-IN GT	116	N/A	1	PROVIDES THE FLOWPATH FOR THE CA TDP TO PUMP FROM THE RC SYSTEM IN A SSS EVENT. OPEN FUNCTION.	0.35	7/12/93	EPRI PPP, MARGIN ANALYSIS, GROUPING	2nd QRT 95
98	2CF0126	6-IN GT	862	N/A	2	SERVES AS PREHEATER BYPASS ISOLATION TO THE AUXILIARY FEEDWATER NOZZLES.	0.5	5/26/89	EPRI PPP, GROUPING	2nd QRT 95
99	2CF0127	6-IN GT	862	N/A	2	SERVES AS PREHEATER BYPASS ISOLATION TO THE AUXILIARY FEEDWATER NOZZLES.	0.5	5/28/89	EPRI PPP, GROUPING	2nd QRT 95
100	2CF0128	6-IN GT	862	N/A	2	SERVES AS PREHEATER BYPASS ISOLATION TO THE AUXILIARY FEEDWATER NOZZLES.	0.5	5/28/89	EPRI PPP, GROUPING	2nd QRT 95

**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

SEQUENCE No.	VALVE NO	VALVE SIZE & TYPE	DESIGN DP PSID	DESIGN FLOW GPM	SAFETY GROUP (1/2)	SAFETY DESCRIPTION	AVAIL. SIZING FACTOR	STATIC TEST DATE	PLANNED STAGE 2 JUSTIFICATION METHODS	PLANNED STAGE 2 COMPLETION DATE
101	2CF0129	6-IN GT	862	N/A	2	SERVES AS PREHEATER BYPASS ISOLATION TO THE AUXILIARY FEEDWATER NOZZLES.	0.5	5/27/89	EPRI PPP, GROUPING	2nd QRT 95
102	2KC0305	3-IN GT	125	N/A	2	TO THE NV EXCESS LETDOWN HEAT EXCHANGER.	0.5	2/23/92	PARTIAL dP TEST	2nd QRT 95
103	2KC0315	3-IN GT	125	N/A	2	TO THE NV EXCESS LETDOWN HEAT EXCHANGER.	0.5	1/29/92	PARTIAL dP TEST	2nd QRT 95
104	2LD0108	4-IN GT	57	N/A	2	BYPASS THE FULL LUBE OIL FILTER ON HIGH DIFFERENTIAL PRESSURE. OPEN FUNCTION.	0.35	2/23/93	OPEN, LARGE MARGIN	2nd QRT 95
105	2LD0113	4-IN GT	57	N/A	2	BYPASS THE FULL LUBE OIL FILTER ON HIGH DIFFERENTIAL PRESSURE. OPEN FUNCTION.	0.35	3/14/93	OPEN, LARGE MARGIN	2nd QRT 95
106	2NC0031	3-IN GT	2335	N/A	1	ISOLATE THE INLET TO EACH PORV	0.5	8/19/93	EPRI PPP, GROUPING	2nd QRT 95
107	2NC0033	3-IN GT	2335	N/A	1	ISOLATE THE INLET TO EACH PORV	0.5	8/19/93	EPRI PPP, GROUPING	2nd QRT 95
108	2NC0035	3-IN GT	2335	N/A	1	ISOLATE THE INLET TO EACH PORV	0.5	8/20/93	EPRI PPP, GROUPING	2nd QRT 95
109	2ND0001	14-IN GT	386	N/A	1	ISOLATE THE NC SYSTEM FROM THE ND SYSTEM.	0.5	7/21/93	EPRI PPP, GROUPING	2nd QRT 95
110	2ND0002	14-IN GT	530	N/A	1	ISOLATE THE NC SYSTEM FROM THE ND SYSTEM. PROVIDE CONTAINMENT ISOLATION DURING A SSS EVENT.	0.5	2/11/92	EPRI PPP, GROUPING	2nd QRT 95
111	2ND0004	14-IN GT	200	N/A	1	PROVIDE ND PUMP SUCTION ISOLATION DURING THE RECIRCULATION PHASE OF AN ECCS ACTUATION.	0.5	7/7/93	EPRI PPP, GROUPING	2nd QRT 95
112	2ND0019	14.00-IN GT	200	N/A	1	PROVIDE ND PUMP SUCTION ISOLATION.	0.5	7/22/93	EPRI PPP, GROUPING	2nd QRT 95
113	2NI0009	4-IN GT	2710	N/A	1	ALLOW HIGH PRESSURE SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM	0.5	2/5/92	EPRI PPP, GROUPING	2nd QRT 95
114	2NI0010	4-IN GT	2710	N/A	1	ALLOW HIGH PRESSURE SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM	0.5	2/6/92	EPRI PPP, GROUPING	2nd QRT 95

**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

SEQUENCE No.	VALVE NO	VALVE SIZE & TYPE	DESIGN DP PSID	DESIGN FLOW GPM	SAFETY GROUP (1/2)	SAFETY DESCRIPTION	AVAIL. SIZING FACTOR	STATIC TEST DATE	PLANNED STAGE 2 JUSTIFICATION METHODS	PLANNED STAGE 2 COMPLETION DATE
115	2NI0054	10-IN GT	633	N/A	1	ALLOW COLD LEG ACCUMULATOR SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM	0.5	7/17/93	EPRI PPP, GROUPING	2nd QRT 95
116	2NI0065	10-IN GT	633	N/A	1	ALLOW COLD LEG ACCUMULATOR SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM	0.5	8/11/93	EPRI PPP, GROUPING	2nd QRT 95
117	2NI0076	10-IN GT	633	N/A	1	ALLOW COLD LEG ACCUMULATOR SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM	0.5	6/18/92	EPRI PPP, GROUPING	2nd QRT 95
118	2NI0088	10-IN GT	633	N/A	1	ALLOW COLD LEG ACCUMULATOR SAFETY INJECTION INTO THE REACTOR COOLANT SYSTEM	0.5	7/17/93	EPRI PPP, GROUPING	2nd QRT 95
119	2NI0100	8-IN GT	185	N/A	1	ALLOW FLOW FROM THE RWST TO THE SIPS. CLOSE ON LOW RWST LEVEL TO ISOLATE RWST FROM THE CCP, SIP, AND RHR PUMPS	0.5	1/23/92	EPRI PPP, GROUPING	2nd QRT 95
120	2NS0001	12-IN GT	68	N/A	1	OPENED TO REALIGN THE SPRAY PUMPS SUCTION FROM THE RWST TO THE CONTAINMENT SUMP.	0.5	7/29/93	SIMILARITY, GROUPING	2nd QRT 95
121	2NS0003	12-IN GT	69	N/A	1	CLOSED TO REALIGN THE SPRAY PUMPS SUCTION FROM THE RWST TO THE CONTAINMENT SUMP.	0.5	8/21/93	SIMILARITY, GROUPING	2nd QRT 95
122	2NS0012	8-IN GT	63	N/A	1	OPEN UPON RECEIPT OF A SP SIGNAL AND STAY OPEN TO PROVIDE FLOW TO THE SPRAY HEADERS.	0.5	7/8/93	SIMILARITY, GROUPING	2nd QRT 95
123	2NS0015	8-IN GT	63	N/A	1	OPEN UPON RECEIPT OF A SP SIGNAL AND STAY OPEN TO PROVIDE FLOW TO THE SPRAY HEADERS.	0.5	7/7/93	SIMILARITY, GROUPING	2nd QRT 95
124	2NS0018	12-IN GT	68	N/A	1	OPENED TO REALIGN THE SPRAY PUMPS SUCTION FROM THE RWST TO THE CONTAINMENT SUMP.	0.5	7/29/93	SIMILARITY, GROUPING	2nd QRT 95
125	2NS0020	12-IN GT	69	N/A	1	CLOSED TO REALIGN THE SPRAY PUMPS SUCTION FROM THE RWST TO THE CONTAINMENT SUMP.	0.5	8/12/93	SIMILARITY, GROUPING	2nd QRT 95



**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

ATTACHMENT 1

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126	2NS0029	8-IN GT	63	N/A	1	OPEN UPON RECEIPT OF A SP SIGNAL AND STAY OPEN TO PROVIDE FLOW TO THE SPRAY HEADERS.	0.5	7/4/93	SIMILARITY, GROUPING	2nd QRT 95
127	2NS0032	8-IN GT	63	N/A	1	OPEN UPON RECEIPT OF A SP SIGNAL AND STAY OPEN TO PROVIDE FLOW TO THE SPRAY HEADERS.	0.5	7/7/93	SIMILARITY, GROUPING	2nd QRT 95
128	2NS0038	8-IN GT	204	N/A	1	OPENED TO PROVIDE A FLOWPATH FROM THE RHR PUMPS TO THE AUXILIARY CONTAINMENT SPRAY HEADERS.	0.5	7/26/93	SIMILARITY, GROUPING	2nd QRT 95
129	2NS0043	8-IN GT	204	N/A	1	THE RHR PUMPS TO THE AUXILIARY CONTAINMENT SPRAY HEADERS.	0.5	7/26/93	SIMILARITY, GROUPING	2nd QRT 95
130	2NV0094	4-IN GT	134	N/A	2	CONTAINMENT ISOLATION FOR THE REACTOR COOLANT PUMP SEAL RETURN.	0.5	7/1/88	SIMILARITY, GROUPING	2nd QRT 95
131	2NV0095	4-IN GT	134	N/A	2	CONTAINMENT ISOLATION FOR THE REACTOR COOLANT PUMP SEAL RETURN.	0.5	7/6/88	SIMILARITY, GROUPING	2nd QRT 95
132	2NV0141	4-IN GT	89	N/A	2	ISOLATION FOR THE VOLUME CONTROL TANK AND THE SUCTION OF THE RECIPROCATING CHARGING PUMPS.	0.5	8/5/93	SIMILARITY, GROUPING	2nd QRT 95
133	2NV0142	4-IN GT	89	N/A	2	AND THE SUCTION OF THE RECIPROCATING CHARGING PUMPS.	0.5	1/30/92	SIMILARITY, GROUPING	2nd QRT 95
134	2NV0221	8-IN GT	183	N/A	1	PROVIDE A FLOWPATH FROM THE RWST TO THE NV PUMPS.	0.5	2/11/92	GROUPING, EPRI PPP	2nd QRT 95
135	2NV0222	8-IN GT	183	N/A	1	PROVIDE A FLOWPATH FROM THE RWST TO THE NV PUMPS.	0.5	8/7/93	GROUPING, EPRI PPP	2nd QRT 95
136	2RN0016	36-IN. BF	8	14500	1	ISOLATION OF RN TRAIN SUPPLY TO RN PUMP.	NA	2EOC9 (12/94)	GROUPING, EPRI PPP	2nd QRT 95
137	2RN0018	36-IN. BF	8	14500	1	ISOLATION OF RN TRAIN SUPPLY TO RN PUMP.	NA	2EOC9 (12/94)	GROUPING, EPRI PPP	2nd QRT 95
138	2RN0042	10-IN. BF	121	98	2	RN SUPPLY TO THE AUX BLDG NON-ESSENTIAL HEADER.	NA	2EOC9 (12/94)	GROUPING, EPRI PPP	2nd QRT 95

**McGUIRE NUCLEAR STATION**  
**MOVs WITHOUT STAGE 2 JUSTIFICATIONS**  
**AS OF 12-28-94**

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139	2RN0279	6-IN. BF	12	750	2	UNIT 2 AUX BLDG VENT UNITS DISCHARGE TO RN SYSTEM.	NA	2EOC9 (12/94)	MARGIN ANALYSIS, EPRI PPP	2nd QRT 95
140	2RN0239	6-IN. BF	12	750	2	UNIT 2 RV AUX BLDG VENT UNITS DISCHARGE TO RN SYSTEM.	NA	2EOC9 (12/94)	MARGIN ANALYSIS, EPRI PPP	2nd QRT 95
141	2RV0032	12-IN. BF	106	4080	2	CONT. ISOL. FOR THE LOWER CONTAINMENT VENT UNITS AND THE INCORE INSTRUMENT ROOM VENT UNITS.	NA	2EOC9 (12/94)	EPRI PPP, GROUPING	2nd QRT 95
142	2RV0033	12-IN. BF	106	4080	2	CONT. ISOL. FOR THE LOWER CONTAINMENT VENT UNITS AND THE INCORE INSTRUMENT ROOM VENT UNITS.	NA	2EOC9 (12/94)	EPRI PPP, GROUPING	2nd QRT 95
143	2RV0076	12-IN. BF	106	4080	2	VENT UNITS AND THE INCORE INSTRUMENT ROOM VENT UNITS.	NA	2EOC9 (12/94)	EPRI PPP, GROUPING	2nd QRT 95
144	2RV0077	12-IN. BF	106	4080	2	VENT UNITS AND THE INCORE INSTRUMENT ROOM VENT UNITS.	NA	2EOC9 (12/94)	EPRI PPP, GROUPING	2nd QRT 95
145	2WL0321	6-IN. BF	91	0	1	CONT. ISOL. FOR THE VUCDT INPUTS.	NA	2EOC9 (12/94)	EPRI PPP, GROUPING	2nd QRT 95
146	2WL0322	6-IN. BF	91	0	1	CONT. ISOL. FOR THE VUCDT INPUTS.	NA	2EOC9 (12/94)	EPRI PPP, GROUPING	2nd QRT 95



## McGUIRE MODIFICATIONS

ATTACHMENT 2

SEQUENCE No.	TAG No.	COMMENTS / IMPROVEMENTS	COMPLETE
1	1CA0042	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
2	1CA0046	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
3	1CA0058	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
4	1CA0062	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
5	2CA0042	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
6	2CA0046	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
7	2CA0058	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
8	2CA0062	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
9	1ND0015	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-43	X
10	1ND0030	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-43	X
11	1NV0221	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
12	2NV0221	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
13	1NV0222	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
14	2NV0222	ACTUATOR CHANGE FROM 16NA-57 TO 16NAI-29	X
15	1CF0126	ACTUATOR CHANGE FROM 30NA-115 TO 30NAI-86	X
16	1CF0127	ACTUATOR CHANGE FROM 30NA-115 TO 30NAI-86	X
17	1CF0128	ACTUATOR CHANGE FROM 30NA-115 TO 30NAI-86	X
18	1CF0129	ACTUATOR CHANGE FROM 30NA-115 TO 30NAI-86	X
19	2CF0126	ACTUATOR CHANGE FROM 30NA-115 TO 30NAI-86	X
20	2CF0127	ACTUATOR CHANGE FROM 30NA-115 TO 30NAI-86	X
21	2CF0128	ACTUATOR CHANGE FROM 30NA-115 TO 30NAI-86	X
22	2CF0129	ACTUATOR CHANGE FROM 30NA-115 TO 30NAI-86	X
23	0RN0003	SPRING PACK CHANGE	X
24	0RN0005	SPRING PACK CHANGE	X
25	0RN0007	SPRING PACK & SMB-000-5 TO SMB-00-10	X
26	0RN0009	SPRING PACK & SMB-000-5 TO SMB-00-10	X
27	0RN0011	SPRING PACK CHANGE	X
28	0RN0013	SPRING PACK CHANGE	X
29	0RN0014	SPRING PACK CHANGE	X
30	0RN0015	SPRING PACK CHANGE	X
31	1RN0016	SPRING PACK CHANGE	X
32	1RN0018	SPRING PACK CHANGE	X

### ATTACHMENT 3

#### LIST OF ABBREVIATIONS

ABBREVIATION	DESCRIPTION
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CA	AUXILIARY FEEDWATER
CCP	CENTRIFUGAL CHARGING PUMP
DBA	DESIGN BASIS ACCIDENT
ECCS	EMERGENCY CORE COOLING SYSTEM
MDP	MOTOR DRIVEN PUMP
NCS, RCS	NUCLEAR COOLANT SYSTEM
ND	DECAY (RESIDUAL) HEAT REMOVAL
NV	CHEMICAL VOLUME CONTROL SYSTEM (CHARGING)
PORV	POWER OPERATED RELIEF VALVE
PRT	PRESSURIZER RELIEF TANK
RC	CONDENSER COOLING WATER
RCS, NCS	REACTOR COOLANT SYSTEM
RHR, ND	DECAY (RESIDUAL) HEAT REMOVAL
RN	SAFETY RELATED RAW WATER
RV	RAW WATER TO CONTAINMENT COOLING
RWST	REFUELING WATER STORAGE TANK
SG, S/G	STEAM GENERATOR
SIPS	SAFETY INJECTION PUMPS
SNSWP	STANDBY NUCLEAR SERVICE WATER POND
SP	SAFETY SIGNAL FOR HIGH CONTAINMENT PRESSURE
SSS	SAFE SHUTDOWN SYSTEM
S/W	SERVICE WATER
TDP	TURBINE DRIVEN PUMP (CA)
VUCDT	VENTILATION UNIT, CONDENSATE DRAIN TANK