

Docket No. 50-336  
B14946

Attachment 1

Millstone Nuclear Power Station, Unit No. 2

Proposed Revision to License Condition and  
Technical Specifications  
Fire Protection

Marked Up Pages

December 1994

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- (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
- (3) Pursuant to the Act and 10 CFR Parts 30, 40, 70, to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensees are authorized to operate the facility at steady-state reactor core power levels not in excess of 2700 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 180 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Fire Protection

INSERT "A"

→ The licensee may proceed with and is required to complete the modifications identified in Section 3 of the NRC's Fire Protection Safety Evaluation on the facility dated September 19, 1978. These modifications shall be completed by the end of the refueling outage presently scheduled for summer 1980.

## INSERT "A"

THE LICENSEE SHALL IMPLEMENT AND MAINTAIN IN EFFECT ALL PROVISIONS OF THE APPROVED FIRE PROTECTION PROGRAM AS DESCRIBED IN THE FINAL SAFETY ANALYSIS REPORT AND AS APPROVED IN THE SER dated SEPTEMBER 19, 1978 AND SUPPLEMENTS DATED NOVEMBER 11, 1981, APRIL 15, 1986, JANUARY 15, 1987, APRIL 29, 1988, AND JULY 17, 1990 subject to the following provisions.

*Y135K  
R1W  
OCTOBER 21, 1980*

*Y135K  
R1W  
OCTOBER 31, 1985*

THE LICENSEE MAY MAKE CHANGES TO THE APPROVED FIRE PROTECTION PROGRAM WITHOUT PRIOR APPROVAL OF THE COMMISSION ONLY IF THOSE CHANGES WOULD NOT ADVERSELY AFFECT THE ABILITY TO ACHIEVE AND MAINTAIN SAFE SHUTDOWN IN THE EVENT OF A FIRE.

May 26, 1988

~~The licensee is required to implement and maintain the administrative controls identified in Section 6 of the NRC's Fire Protection Safety Evaluation on the facility dated September 19, 1978. The administrative controls shall be in effect by December 31, 1978.~~

(4) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training, and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 CFR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Millstone Nuclear Power Station Physical Security Plan," with revisions submitted through March 29, 1988; "Millstone Nuclear Power Station Suitability, Training, and Qualification Plan," with revision submitted through July 21, 1986; and "Millstone Nuclear Power Station Safeguards Contingency Plan," with revisions submitted through October 30, 1985. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

- D. This amended license is effective as of its date of issuance and shall expire at midnight July 31, 2015.

FOR THE NUCLEAR ENERGY COMMISSION

Roger S. Boyd, Acting Director  
Division of Reactor Licensing  
Office of Nuclear Reactor Regulation

Original Signed by Roger S. Boyd

Enclosures:

1. Incomplete Preoperational Test  
Items Which Must be Completed
2. Change No 4 to Technical  
Specifications Contained in  
Appendix A to DPR-65.

Date of Issuance: September 26, 1975

Amendment No. 129



## INDEX

### LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.2 POWER DISTRIBUTION LIMITS</u>	
3/4.2.1 LINEAR HEAT RATE.....	3/4 2-1
3/4.2.2 Deleted	
3/4.2.3 TOTAL INTEGRATED RADIAL PEAKING FACTOR - $F_r^T$ .....	3/4 2-9
3/4.2.4 AZIMUTHAL POWER TILT.....	3/4 2-10
3/4.2.5 Deleted	
3/4.2.6 DNB MARGIN.....	3/4 2-13
<u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 REACTOR PROTECTIVE INSTRUMENTATION.....	3/4 3-1
3/4.3.2 ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION.....	3/4 3-10
3/4.3.3 MONITORING INSTRUMENTATION.....	3/4 3-26
Radiation Monitoring.....	3/4 3-26
Incore Detectors.....	3/4 3-30
Seismic Instrumentation.....	3/4 3-32
Meteorological Instrumentation.....	3/4 3-36
Chlorine Detection Systems.....	3/4 3-42
<del>Fire Detection Instrumentation.....</del>	<del>3/4 3-43</del>
Accident Monitoring.....	3/4 3-46
Radioactive Liquid Effluent Monitoring Instrumentation.	3/4 3-50
Radioactive Gaseous Effluent Monitoring Instrumentation	3/4 3-56
 <u>3.4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1 COOLANT LOOPS AND COOLANT CIRCULATION.....	3/4 4-1
Startup and Power Operation.....	3/4 4-1
Hot Standby.....	3/4 4-1a
Shutdown.....	3/4 4-1b

INDEXLIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.7 PLANT SYSTEMS</u>	
3/4.7.1 TURBINE CYCLE .....	3/4 7-1
Safety Valves .....	3/4 7-1
Auxiliary Feedwater Pumps .....	3/4 7-4
Condensate Storage Tank .....	3/4 7-6
Activity .....	3/4 7-7
Main Steam Line Isolation Valves .....	3/4 7-9
3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION .....	3/4 7-10
3/4.7.3 REACTOR BUILDING CLOSED COOLING WATER SYSTEM .....	3/4 7-11
3/4.7.4 SERVICE WATER SYSTEM .....	3/4 7-12
3/4.7.5 FLOOD LEVEL .....	3/4 7-13
3/4.7.6 CONTROL ROOM EMERGENCY VENTILATION SYSTEM .....	3/4 7-16
3/4.7.7 SEALED SOURCE CONTAMINATION .....	3/4 7-19
3/4.7.8 SNUBBERS .....	3/4 7-21
3/4.7.9 FIRE SUPPRESSION SYSTEMS .....	<del>3/4 7-33</del>
Fire Suppression Water System .....	<del>3/4 7-33</del>
Spray and/or Sprinkler Systems .....	<del>3/4 7-37</del>
Fire Hose Stations .....	<del>3/4 7-39</del>
Halon Fire Suppression Systems .....	<del>3/4 7-41</del>
3/4.7.10 PENETRATION FIRE BARRIERS .....	<del>3/4 7-42</del>
3/4.7.11 ULTIMATE HEAT SINK .....	3/4 7-44 <sup>34</sup>
 <u>3/4.8 ELECTRICAL POWER SYSTEMS</u>	
3/4.8.1 A.C. SOURCES .....	3/4 8-1
Operating .....	3/4 8-1
Shutdown .....	3/4 8-5
3/4.8.2 ONSITE POWER DISTRIBUTION SYSTEMS .....	3/4 8-6
A.C. Distribution - Operating .....	3/4 8-6
A.C. Distribution - Shutdown .....	3/4 8-7
D.C. Distribution - Operating .....	3/4 8-8
D.C. Distribution - Shutdown .....	3/4 8-10

DELETED

# INDEX

## BASES

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.7 PLANT SYSTEMS</u>	
3/4.7.1 TURBINE CYCLE .....	B 3/4 7-1
3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION .....	B 3/4 7-3
3/4.7.3 REACTOR BUILDING CLOSED COOLING WATER SYSTEM .....	B 3/4 7-3
3/4.7.4 SERVICE WATER SYSTEM .....	B 3/4 7-4
3/4.7.5 FLOOD LEVEL .....	B 3/4 7-4
3/4.7.6 CONTROL ROOM EMERGENCY VENTILATION SYSTEM .....	B 3/4 7-4
3/4.7.7 SEALED SOURCE CONTAMINATION .....	B 3/4 7-5
3/4.7.8 SNUBBERS .....	B 3/4 7-5
<del>3/4.7.9 FIRE SUPPRESSION SYSTEMS .....</del>	<del>B 3/4 7-6</del>
<del>3/4.7.10 PENETRATION FIRE BARRIERS .....</del>	<del>B 3/4 7-7</del>
3/4.7.11 ULTIMATE HEAT SINK .....	B 3/4 7-7
 <u>3/4.8 ELECTRICAL POWER SYSTEMS</u> .....	 B 3/4 8-1
<u>3/4.9 REFUELING OPERATIONS</u>	
3/4.9.1 BORON CONCENTRATION .....	B 3/4 9-1
3/4.9.2 INSTRUMENTATION .....	B 3/4 9-1
3/4.9.3 DECAY TIME .....	B 3/4 9-1
3/4.9.4 CONAINMENT PENETRATIONS .....	B 3/4 9-1
3/4.9.5 COMMUNICATIONS .....	B 3/4 9-1
3/4.9.6 CRANE OPERABILITY--CONTAINMENT BUILDING .....	B 3/4 9-2
3/4.9.7 CRANE TRAVEL--SPENT FUEL STORAGE BUILDING .....	B 3/4 9-2
3/4.9.8 SHUTDOWN COOLING AND COOLING RECIRCULATION .....	B 3/4 9-2

## INSTRUMENTATION

### ~~FIRE DETECTION INSTRUMENTATION~~

#### ~~LIMITING CONDITION FOR OPERATION~~

~~3.3.3.7 As a minimum, the fire detection instrumentation for each fire detection zone shown in Table 3.3-10 shall be OPERABLE.~~

~~APPLICABILITY: Whenever equipment in that fire detection zone is required to be OPERABLE.~~

#### ~~ACTION:~~

~~With the number of OPERABLE fire detection instrument(s) less than the minimum number of OPERABLE requirements of Table 3.3-10:~~

- ~~a. Within 1 hour establish a fire watch patrol to inspect the zone(s) with the inoperable instrument(s) at least once per hour unless the instrument(s) is located inside the containment, then inspect the containment at least once per 8 hours or monitor the containment air temperatures at least once per hour at the locations listed in Specification 4.6.1.5.~~
- ~~b. Restore the inoperable instrument(s) to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.6.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the instrument(s) to OPERABLE status.~~
- ~~c. The provisions of Specification 3.0.3 are not applicable.~~

#### SURVEILLANCE REQUIREMENTS

~~4.3.3.7.1 Each of the above required fire detection instruments which are accessible during plant operation shall be demonstrated OPERABLE at least once per 6 months by performance of a CHANNEL FUNCTIONAL TEST. Fire detectors which are not accessible during plant operation shall be demonstrated OPERABLE by the performance of a CHANNEL FUNCTIONAL TEST during each COLD SHUTDOWN exceeding 24 hours unless performed in the previous 6 months.~~

~~4.3.3.7.2 The circuitry associated with the supervision of the above fire detection instruments and circuits, per NFPA 72-D, shall be demonstrated OPERABLE at least once per 6 months.~~

~~4.3.3.7.3 The nonsupervised circuits, associated with detector alarms, between the instrument and the control room shall be demonstrated OPERABLE at least once per 31 days.~~

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TABLE 3.3-10

## FIRE DETECTION INSTRUMENTS

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Instrument Location (Zone)	Heat		Smoke	
	Total No. of Channels	Minimum Channels Operable	Total No. of Channels	Minimum Channels Operable
1. Containment				
East Penetration (37)--FLP 3-3	--	--	7	5
West Penetration (31)--FLP 3-7	--	--	7	5
2. Control Room Vent Duct (42) Z-2	--	--	1	1
Control Room Vent Duct (2) Z-1	--	--	1	1
3. Cable Vaults & Areas				
Aux. Bldg Cable Vault (25') (10)	5	4	16	12
Turbine Bldg. Cable Vault (25') (22)	--	--	34	34
Turbine Bldg Cable Vault Area (45') (21)	--	--	8	6
Lunch Room Cable Chase Area (36'6") (24)	--	--	4	3
4. 4.16 & 6.9 kV Switchgear Room (54'6") (40)	--	--	4	3
4.16 kV Switchgear Room (31'6") (18)	--	--	4	3
480 V West Switchgear Room (36'6") (18)	--	--	2	1
480 V East Switchgear Room (36'6") (28)	--	--	2	1
East DC Equipment Room (43 Alarm) (FLP-5)	--	--	6	6
West DC Equipment Room (45 Alarm) (FLP-6)	--	--	6	6
East Cable Vault Ventilation Opening (44) (FLP 7)	--	--	1	1
West Cable Vault Ventilation Opening (44) (FLP 7)	--	--	1	1
5. Battery Rooms				
West Battery Room (14'6") (39)	--	--	1	1
East Battery Room (14'6") (39)	--	--	2	1
6. Electrical Penetration Rooms				
East (14'6") (20)	--	--	3	2
West (14'6") (17)	--	--	2	1

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MILLSTONE - UNIT 2

3/4 3-44

Amendment No. 28, 27, 110, 152

August 26, 1992

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~~TABLE 3 - 10 (Continued)~~

~~FIRE DETECTION INSTRUMENTS~~

<u>Instrument Location (Zone)</u>	<u>Heat</u>		<u>Smoke</u>	
	<u>Total No. of Channels</u>	<u>Minimum Channels Operable</u>	<u>Total No. of Channels</u>	<u>Minimum Channels Operable</u>
7. Diesel Generators				
Diesel 1221 (12)	8	8	--	--
Diesel 1321 (13)	8	8	--	--
8. Main Exhaust Equipment Room and B52 Enclosure				
Room (E1 38'6") (5)	--	--	3	3
9. Auxiliary Building - 45 (FLP-1)				
General Area (48)	--	--	3	2
A. Safe Guards Room (48)	--	--	2	1
B. Safe Guards Room (48)	--	--	2	1
C. Safe Guards Room (48)	--	--	1	1
10. Auxiliary Building - 25 (FLP-2)				
General Area - 25 (52)	--	--	9	7
Charging Pump Rooms - 25 (52)	--	--	5	3
11. Containment Building FLP-3 (37)				
RCP "A" - (FLP 3-1)	5	3	--	--
RCP "B" - (FLP 3-2)	5	3	--	--
RCP "C" - (FLP 3-5)	5	3	--	--
RCP "D" - (FLP 3-6)	5	3	--	--
12. Auxiliary Building (-5'/14'6") (FLP-4)				
Auxiliary Building General Area 14'6" (41)	--	--	9	7
Auxiliary Building West Piping Penetration (41)	--	--	2	1
Room -5'	--	--	13	10
Auxiliary Building -5' (41)	--	--		

MILLSTONE - UNIT 2

3/4 3-45

Amendment No. 25, 27, 110, 112



~~TABLE 3.3-10 (Continued)~~  
~~FIRE DETECTION INSTRUMENTS~~

<u>Instrument Location (Zone)</u>	<u>Heat</u>		<u>Smoke</u>	
	<u>Total No. of Channels</u>	<u>Minimum Channels Operable</u>	<u>Total No. of Channels</u>	<u>Minimum Channels Operable</u>
13. Hydrogen Seal Oil (31)	6	6	--	--
14. Intake Structure (6)	--	--	10	10
15. Motor Driven Aux. Feed Pump Rm (22)	--	--	2	2

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3/4 3-45a

Amendment No. 162

PLANT SYSTEMS

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3.7.9.1 FIRE SUPPRESSION SYSTEMS

FIRE SUPPRESSION WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.9.1 The fire suppression water system shall be OPERABLE with:

- a. Three high pressure pumps, each with a capacity of at least 1800 gpm, with their discharge aligned to the fire suppression header.
- b. Two water supplies, each with a minimum contained volume of 200,000 gallons, and
- c. An OPERABLE flow path capable of taking suction from the fire water tanks and transferring the water through distribution piping with OPERABLE sectionalizing control or isolation valves to the yard hydrant curb valves and the first valve ahead of the water flow alarm device on each sprinkler, hose standpipe or spray system riser required to be OPERABLE per Specifications 3.7.9.2 and 3.7.9.3.

APPLICABILITY: At all times.

ACTION:

- a. With one pump and/or one water supply inoperable, restore the inoperable equipment to OPERABLE status within 7 days or, in lieu of any other report required by Specification 6.6.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the plans and procedures to be used to provide for the loss of redundancy in this system. The provisions of Specification 3.0.3 are not applicable.
- b. With two pumps inoperable, establish a continuous fire watch of the turbine building with backup fire suppression equipment within 1 hour; restore the inoperable equipment to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the equipment to OPERABLE status. The provisions of Specification 3.0.3 are not applicable.

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PLANT SYSTEMS

LIMITING CONDITION FOR OPERATION (Continued)

- c. With the fire suppression water system otherwise inoperable:
1. Establish a backup fire suppression water system within 24 hours, and
  2. Submit a Special Report in accordance with Specification 6.9.2:
    - a) By telephone within 24 hours,
    - b) Confirmed by telegraph, mailgram or facsimile transmission no later than the first working day following the event, and
    - c) In writing within 14 days following the event, outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

SURVEILLANCE REQUIREMENTS

- 4.7.9.1.1 The fire suppression water system shall be demonstrated OPERABLE:
- a. At least once per 7 days by verifying the contained water supply volume.
  - b. At least once per 31 days on a STAGGERED TEST BASIS by starting each pump and operating it for at least 15 minutes on recirculation flow.
  - c. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path is in its correct position.
  - d. At least once per 12 months by cycling each testable valve in the flow path through at least one complete cycle of full travel.

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PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- e. At least once per 18 months by performing a system functional test which includes simulated automatic actuation of the system throughout its operating sequence, and:
  - 1. Verifying that each pump develops at least 1800 gpm at a system head of 100 psig.
  - 2. Cycling each valve in the flow path that is not testable during plant operation through at least one complete cycle of full travel, and
  - 3. Verifying that each high pressure pump starts (sequentially) to maintain the fire suppression water system pressure  $\geq 75$  psig.
- f. At least once per 3 years by performing a flow test of the system in accordance with Chapter 5, Section 11 of the Fire Protection Handbook, 14th Edition, published by the National Fire Protection Association.

4.7.9.1.2 The fire pump diesel engine shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying:
  - 1. The fuel storage tank contains at least 125 gallons of fuel, and
  - 2. The diesel starts from ambient conditions and operates for at least 20 minutes.
- b. At least once per 92 days by verifying that a sample of diesel fuel from the fuel storage tank, obtained in accordance with ASTM-D270-65, is within the acceptable limits specified in Table Y of ASTM D975-74 when checked for viscosity, water and sediment.
- c. At least once per 18 months by:
  - 1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for the class of service, and
  - 2. Verifying the diesel starts from ambient conditions on the auto-start signal and operates for  $\geq 20$  minutes while loaded with the fire pump.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.7.9.1.3 The fire pump diesel starting 12-volt batteries and charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
  - 1. The electrolyte level of each battery is above the plates, and
  - 2. The overall battery voltage is  $\geq 12$  volts.
- b. At least once per 92 days by verifying that the specific gravity is appropriate for continued service of the batteries.
- c. At least once per 18 months by verifying that:
  - 1. The batteries, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration, and
  - 2. The terminal connections are clean, tight, free of corrosion and coated with anti-corrosion material.

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PLANT SYSTEMS~~SPRAY AND/OR SPRINKLER SYSTEMS~~~~LIMITING CONDITION FOR OPERATION~~

3.7.9.2 The following spray and/or sprinkler systems shall be OPERABLE:

- a. Diesel Generator Rooms
- b. Diesel Generator Day Tank Rooms
- c. Cable Vault (Aux. Building)
  - 1. Sprinkler (in tray)
  - 2. Sprinkler (ceiling level)
- d. Cable Vault (Turbine Building)
  - 1. Automatic Wet Pipe Sprinkler System (45'0")
  - 2. Automatic Deluge System (vertical cable shaft and elevation 25'0" cable vault)
- e. Hydrogen Seal Oil Unit
- f. Turbine Building Northeast Corner
- g. Turbine Building 31'6"/14'6" - North
- h. Turbine Building 31'6"/14'6" - South
- i. Lube Oil Room
- j. Aux. Building (-45'6") General Area
- k. Aux. Building (14'6") Truck Access
- l. Turbine Bearing
- m. Steam Generator Feed Pumps
- n. Aux. Bldg. (14'6") at MCC B-61
- o. Aux. Bldg. (-25'6") at Charging Pump Cubicle
- p. Aux. Bldg. (14'6") General Area

APPLICABILITY: Whenever equipment in the spray/sprinkler protected areas is required to be OPERABLE.

ACTION:

- a. With one or more of the above required spray and/or sprinkler systems inoperable, establish a continuous fire watch with backup fire suppression equipment for the unprotected area(s) within 1 hour; restore the system to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.6.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENT

4.7.9.2 Each of the above required spray and/or sprinkler systems shall be demonstrated OPERABLE:

MILLSTONE - UNIT 2  
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3/4 7-37

Amendment No. 75, 77, 110, 111, 191, 162

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SURVEILLANCE REQUIREMENTS (Continued)

- a. At least once per 12 months by cycling each testable valve in the flow path through at least one complete cycle of full travel.
- b. At least once per 18 months:
  - 1. By performing a system functional test which includes simulated automatic actuation of the system; and:
    - a) Verifying that the automatic valves in the flow path actuate to their correct positions on a simulated test signal, and
    - b) Cycling each valve in the flow path that is not testable during plant operation through at least one complete cycle of full travel.
  - 2. By inspection of the spray headers to verify their integrity, and
  - 3. By inspection of each nozzle to verify no blockage.
- c. At least once per 3 years by performing an air or water flow test through each open head spray/sprinkler header and verifying each open head spray/ sprinkler nozzle is unobstructed.

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PLANT SYSTEMS

~~FIRE HOSE STATIONS~~

~~LIMITING CONDITION FOR OPERATION~~

3.7.9.3 ~~The fire hose stations shown in Table 3.7-2 shall be OPERABLE.~~

APPLICABILITY: Whenever equipment in the areas protected by the fire hose stations is required to be OPERABLE.\*

ACTION

- a. With one or more of the fire hose stations shown in Table 3.7-2 inoperable, route an additional equivalent capacity fire hose to the unprotected area(s) from an OPERABLE hose station within 1 hour or establish a continuous fire watch with backup fire suppression equipment for the unprotected area(s). If the inoperable hose station(s) is not the primary means of fire suppression, then route the additional fire hose(s) or establish a continuous fire watch with fire suppression equipment within 24 hours.
- b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.9.3 Each of the fire hose stations shown in Table 3.7-2 shall be demonstrated OPERABLE:

- a. At least once per 31 days by visual inspection of the station to assure all required equipment is at the station. The exception to the above will be the containment hose stations. The equipment will be located outside containment except when the unit is in cold shutdown.
- b. At least once per 18 months by:
  1. Removing the hose for inspection and re-racking, and
  2. Replacement of all degraded gaskets in couplings.
- c. At least once per 3 years by:
  1. Partially opening each hose station valve to verify valve OPERABILITY and no flow blockage.
  2. Conducting a hose hydrostatic test at a pressure at least 50 psig greater than the maximum pressure available at that hose station.

\*Containment hose stations shall be operable in MODE 5 when required to support maintenance activities and MODE 6.

TABLE 3.7-2

~~FIRE HOSE STATIONS~~

DELETED

<u>Hose Station Number</u>	<u>Bldg/Elevation</u>	<u>Area</u>
201-207	Turbine/14'6"	Turbine Building
208-214	Turbine/31'6"	Turbine Building
215-221	Turbine/54'6"	Turbine Building
222	Auxiliary/-45'6"	Center of Open Area
223	Auxiliary/-25'6"	Near Elevator
224	Auxiliary/-5'0"	Near Elevator
225	Auxiliary/14'6"	Near Elevator
226	Auxiliary/38'6"	Spent Fuel Pool - Northwest corner
227	Auxiliary/14'6"	Boric Acid Batch Tank area
228	Auxiliary/14'6"	Near MCC 22-1E (B51)
229	Auxiliary/14'6"	Railway access
230	Auxiliary/38'6"	Spent Fuel Pool - South Wall
231	Auxiliary/14'6"	Outside Diesel Room
234	Auxiliary/38'6"	Southeast corner stairway
240	Auxiliary/5'0"	Southeast corner stairway
241	Auxiliary/25'6"	Cable Vault Southeast Entrance
242	Auxiliary/36'6"	Control Room Ventilation Area
243	Turbine/45'0"	North Entrance of Turbine Bldg. Cable Vaults
250-251	Containment/38'6"	East & West Stairwells
248-249	Containment/14'6"	East & West Stairwells
244-245	Containment/-22'0"	East & West Stairwells

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PLANT SYSTEMSHALON FIRE SUPPRESSION SYSTEMLIMITING CONDITION FOR OPERATION

*DELETED*  
3.7.9.4 The following halon 1301 fire suppression systems shall be OPERABLE with: an intact gas boundary, an operable activation system, and a container having a net weight of not less than 95% of full charge weight at 325 psig minimum (corrected to 70°F).

- a) West D.C. Switchgear Room
- b) East D.C. Switchgear Room

APPLICABILITY: At all times.

ACTION:

- a. With one or more of the above systems inoperable, establish a continuous fire watch with backup fire suppression equipment for the unprotected area(s) within 1 hour; restore the system to OPERABLE status within 14 days or, in lieu of any other report required by Specification 6.6.1, prepare and submit a Special Report to the Commission, pursuant to Specification 6.9.2, within the next 30 days outlining the action taken the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.9.4 Each of the above halon fire suppression systems shall be demonstrated OPERABLE:

- a. At least once per 6 months:
  - 1. By performing a system functional test which includes simulated automatic operation of the system; and:
    - a. simulated manual actuation of the system.
    - b. verifying that the storage containers have a net weight of not less than 95% of full charge weight at 325 psig (corrected to 70°F).
    - c. verifying the associated room dampers close.
- b. At least once per 18 months:
  - 1. By performing a visual inspection of the discharge nozzles to assure no blockage.
  - 2. By performing a visual inspection to assure the gas boundary is intact.

PLANT SYSTEMS~~DELETED~~3/4.7.10 PENETRATION FIRE BARRIERS

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LIMITING CONDITIONS FOR OPERATION

3.7.10 All fire rated assemblies (walls, floor/ceilings, cable tray enclosures, and other fire barriers) separating safety-related fire areas or separating portions of redundant systems important to safe shutdown within a fire area and all sealing devices in fire rated assembly penetrations (fire doors, fire windows, fire dampers, cable, piping, and ventilation duct penetration seals) shall be OPERABLE.

APPLICABILITY: At all times unless otherwise determined that the separation of safety-related fire areas or separating portions of redundant systems important to safe shutdown within a fire area is not required based on the MODE of operation.

ACTION:

- a. With one or more of the above required fire rated assemblies and/or penetration sealing devices inoperable, within 1 hour:
  1. Determine that the fire areas/zones on both sides of the affected fire rated assembly and/or penetration sealing device are monitored by either an OPERABLE fire detection or automatic suppression system at the fire barrier and establish a fire watch patrol that inspects both areas at least once per hour, or
  2. Establish a continuous fire watch on at least one side of the affected fire rated assembly and/or penetration seal, or
  3. Temporarily repair the inoperable fire rated assembly and/or sealing device and classify it as temporary.

All temporary or inoperable fire rated assemblies and/or sealing devices shall be permanently repaired within 30 days, or implement ACTION 1 or 2 above.

- b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.10 The above required fire rated assemblies and penetration sealing devices shall be verified to be OPERABLE by a visual inspection:

- a. At least once per 18 months for fire doors and fire dampers.



SURVEILLANCE REQUIREMENTS (Continued)

- b. At least once per 18 months for fire barrier penetration seals, on at least 10% of the total number of penetration seals. If any of the penetration seals in the inspection sample are found to be inoperable, then an additional 10% sample of the total number of penetration seals shall be visually inspected. Sampling and inspection shall continue until all of the seals in a sample are found OPERABLE or 100% of the seals are inspected.
- c. Prior to returning a fire rated assembly and/or penetration sealing device to OPERABLE status following repairs or maintenance.

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PLANT SYSTEMS3/4.7.11 ULTIMATE HEAT SINKLIMITING CONDITION FOR OPERATION

3.7.11 The ultimate heat sink shall be OPERABLE with an average water temperature of less than or equal to 75°F at the Unit 2 intake structure.

APPLICABILITY: MODES 1, 2, 3, AND 4

ACTION:

With the requirements of the above specification not satisfied, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.11 The ultimate heat sink shall be determined OPERABLE:

- a. At least once per 24 hours by verifying the average water temperature at the Unit 2 intake structure to be within limits.
- b. At least once per 6 hours by verifying the average water temperature at the Unit 2 intake structure to be within limits when the average water temperature exceeds 70°F.

## INSTRUMENTATION

**DELETED**

### 3/4.3.3.6 Fire Detection Instrumentation

~~OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is required in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety related equipment and is an integral element in the overall facility fire protection program.~~

~~In the event that a portion of the fire detection instrumentation is inoperable, the establishment of frequent fire patrols in the affected areas is required to provide detection capability until the inoperable instrumentation is restored to OPERABILITY.~~

3/4.3.3.7

### 3/4.3.3.7 Accident Monitoring Instrumentation

~~The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of NUREG-0578, "TM1-2 Lessons Learned Task Force Status Report and Short-Term Recommendations".~~

Deleted

## PLANT SYSTEMS

### BASES

When the cause of the rejection of a snubber is clearly established and remedied for that snubber and for any other snubbers that may be generically susceptible, that snubber may be exempted from being counted as inoperable. Generically susceptible snubbers are those which are of a specific make or model and have the same design features directly related to rejection of the snubber by visual inspection, or are similarly located or exposed to the same environmental conditions such as temperature, radiation, and vibration. Due to the size and location of the steam generator hydraulic snubbers, regular removal and testing as specified for hydraulic and mechanical snubbers would represent a significant undertaking during each refueling outage. As such, these snubbers have been treated separately and are tested and refurbished as a group in accordance with the manufacturer's recommended preventative maintenance program.

When a snubber is found inoperable, an engineering evaluation is performed, in addition to the determination of the snubber mode of failure, in order to determine if any safety-related component or system has been adversely affected by the inoperability of the snubber.

The engineering evaluation shall determine whether or not the snubber mode of failure has imparted a significant effect or degradation on the supported component or system.

To provide assurance of snubber reliability, a representative sample of the installed snubbers will be tested during plant shutdowns at eighteen (18) month intervals. Observed failures of these sample snubbers shall require testing of additional units.

Hydraulic snubbers and mechanical snubbers may each be treated as a different entity for the above surveillance programs.

The service life of a snubber is evaluated via manufacturer input and information through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubber, seal replaced, spring replaced, in high radiation area, in high temperature area, etc....). The requirement to monitor the snubber service life is included to ensure that the snubbers periodically undergo a performance evaluation in view of their age and operating conditions. These records will provide statistical bases for future consideration of snubber service life. The requirements for the maintenance of records and the snubber service life review are not intended to affect plant operation.

DELETED

### 3/4.7.9 FIRE SUPPRESSION SYSTEMS

~~The OPERABILITY of the fire suppression system ensures that adequate fire suppression capability is available to confine and extinguish fires occurring in any portion of the facility where safety related equipment is located. The fire~~

PLANT SYSTEMSBASES

Suppression system consists of the water system, spray and/or sprinklers and fire hose stations. The collective capability of the fire suppression systems is adequate to minimize potential damage to safety related equipment and is a major element in the facility fire protection program.

In the event that portions of the fire suppression systems are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the inoperable equipment is restored to service.

In the event the fire suppression water system becomes inoperable, immediate corrective measures must be taken since this system provides the major fire suppression capability of the plant. The requirement for a twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability for the continued protection of the nuclear plant.

#### 3.4.7.10 PENETRATION FIRE BARRIERS

**DELETED**

The functional integrity of the penetration fire barriers ensures that fires will be confined or adequately retarded from spreading to adjacent portions of the facility. This design feature minimizes the possibility of a single fire rapidly involving several areas of the facility prior to detection and extinguishment. The penetration fire barriers are a passive element in the facility fire protection program and are subject to periodic inspections.

During period of time when a barrier is not functional, alternate measures are taken to prevent the possible spread of fire. These measures include verifying the operability of fire detection or suppression systems on both sides of the affected barrier and establishing a fire watch patrol, or posting a continuous fire watch in the vicinity of the affected barrier, or installation of a temporary fire stop pending restoration of the permanent seal.

#### 3.4.7.11 ULTIMATE HEAT SINK

The limitations on the ultimate heat sink temperature ensure that sufficient cooling capacity is available to either,

- 1) provide normal cooldown of the facility, or 2) to mitigate the effects of accident conditions within acceptable limits.

The limitations on maximum temperature are based on a 30-day cooling water supply to safety related equipment without exceeding their design basis temperature.

ADMINISTRATIVE CONTROLSFACILITY STAFF (CONTINUED)

- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor. (Table 6.2-1)
- e. ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. (Table 6.2-1) The Fire Brigade shall not include 2 members of the minimum shift crew necessary for safe shutdown of the unit or any personnel required for other essential functions during a fire emergency.

- g. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions. These procedures should follow the general guidance of the NRC Policy Statement on working hours (Generic Letter No. 82-12).

6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for:

- a. If the Operations Manager does not hold a senior reactor operator license for Millstone Unit No. 2, then the Operations Manager shall have held a senior reactor operator license at a Pressurized Water Reactor other than Millstone Unit No. 2 and an individual serving in the capacity of the Assistant Operations Manager shall hold a senior reactor operator license for Millstone Unit No. 2.

- b. The Shift Technical Advisor (STA) who shall meet the requirements of Specification 6.3.1.b.1 or 6.3.1.b.2.

- 1. Dual-role individual: Must hold a senior reactor operator's license at Millstone Unit No. 2, meet the STA training criteria of NUREG-0737, Item I.A.1.1, and meet one of the following educational alternatives:

- a. Bachelor's degree in engineering from an accredited institution;
- b. Professional Engineer's license obtained by the successful completion of the PE examination;



TABLE 6.2-1<sup>(3)</sup>  
MINIMUM SHIFT-CREW COMPOSITION<sup>(2)</sup>

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
Senior Reactor Operator	2	1 <sup>(1)</sup>
Reactor Operator	2	1
Non-Licensed Operator	2	1
Shift Technical Advisor	1 <sup>(4)</sup>	None Required

- (1) Does not include the licensed Senior Reactor or Senior Reactor Operator Limited to Fuel Handling individual supervision CORE ALTERATIONS after the initial fuel loading.
- (2) The above shift crew composition and the qualified health physics technician ~~and/or fire brigade members~~ of Section 6.2.2 may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided expeditious actions are taken to fill the required positions.
- (3) Requirements for minimum number of licensed operators on shift during operation in modes other than cold shutdown or refueling are contained in 10CFR50.54(m).
- (4) The Shift Technical Advisor position can be filled by either of the two Senior Reactor Operators (a dual-role individual), if he meets the requirements of Specification 6.3.1.b.1.



## ADMINISTRATIVE CONTROLS

### 6.4 TRAINING

- ~~6.4.1~~ A retraining and replacement training program for the facility staff shall be maintained under the direction of the Senior Vice President — Millstone Station and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.59. The Director-Nuclear Training has the overall responsibility for the implementation of the Training Program.
- ~~6.4.2~~ A training program for the Fire Brigade shall be maintained under the direction of the Director-Nuclear Training and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for Fire Brigade training sessions which shall be held at least quarterly.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 Plant Operations Review Committee (PORC)

##### Function

- 6.5.1.1 The PORC shall function to advise the Nuclear Unit Director on all matters related to nuclear safety.

##### Composition

- 6.5.1.2 The PORC shall be composed of the:

Chairperson:	Nuclear Unit Director
Vice Chairperson & Member:	Operations Manager
Member:	Maintenance Manager
Member:	Instrument and Controls Manager
Member:	Reactor Engineer
Member:	Radiation Protection Supervisor or Chemistry Supervisor
Member:	Engineering Manager
Member:	Staff Engineer

##### Alternates

- 6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairperson to serve on a temporary basis; however, no more than two alternates shall participate in PORC activities at any one time.

ADMINISTRATIVE CONTROLSMeeting Frequency

- 6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairperson.

Quorum

- 6.5.1.5 A quorum of the PORC shall consist of the Chairperson, or Vice Chairperson, or Senior Vice President — Millstone Station and four members including alternates.

Responsibilities

- 6.5.1.6 The PORC shall be responsible for:

- a. Review of 1) all procedures, except common site procedures, required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Nuclear Unit Director to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Sections 1.0 - 5.0 of these Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Executive Vice President-Nuclear and to the Chairperson of the Nuclear Review Board.
- f. Review of all REPORTABLE EVENTS.
- g. Review of facility operations to detect potential safety hazards.
- h. Performance of special reviews and investigations and reports thereon as requested by the Chairperson of the Nuclear Review Board.
- i. Render determinations in writing if any item considered under 6.5.1.6(a) through (d) above, as appropriate and as provided by 10CFR50.59 or 10CFR50.92, constitutes an unreviewed safety question or requires a significant hazards consideration determination.

J. Review of the fire protection program and implementing Procedure.

ADMINISTRATIVE CONTROLSMeeting Frequency

- 6.5.2.4 The SORC shall meet at least once per six months and as convened by the SORC Chairperson.

Quorum

- 6.5.2.5 A quorum of the SORC shall consist of the Chairperson or Vice Chairperson and five members including alternates.

Responsibilities

- 6.5.2.6 The SORC shall be responsible for:

- a. Review of 1) all common site procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Senior Vice President — Millstone Station to affect site nuclear safety.
- b. Review of all proposed changes to Section 6.0 "Administrative Controls" of these Technical Specifications.
- c. Performance of special reviews and investigations and reports as requested by the Chairperson of the Site Nuclear Review Board.
- d. Review of the Plant Security Plan and implementing procedures and shall submit changes to the Chairperson of the Site Nuclear Review Board.
- e. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairperson of the Site Nuclear Review Board.
- f. Review of all common site proposed tests and experiments that affect nuclear safety.
- g. Review of all common site proposed changes or modifications to systems or equipment that affect nuclear safety.
- h. Render determinations in writing or meeting minutes if any item considered under 6.5.2.6(a) through (g) above, as appropriate and as provided by 10CFR50.59 or 10CFR50.92, constitutes an unreviewed safety question or requires a significant hazards consideration determination.

Authority

- i. Review of the common site fire protection program and implementing procedures.

- 6.5.2.7 The SORC shall:

- a. Recommend to the Senior Vice President— Millstone Station written approval or disapproval in meeting minutes of items considered under 6.5.2.6(a) through (g) above.

# ADMINISTRATIVE CONTROLS

## SPECIAL REPORTS (CONT.)

- b. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- c. Safety Class 1 Inservice Inspection Program Review, Specification 4.4.10.1.
- d. ECCS Actuation, Specifications 3.5.2 and 3.5.3.
- e. ~~Fire Detection Instrumentation, Specifications 3.3.3.7.~~
- f. ~~Fire Suppression Systems, Specifications 3.7.9.1, 3.7.9.2, and 3.7.9.4.~~
- g. RCS Overpressure Mitigation, Specification 3.4.9.3.
- h. Radiological Effluent Reports required by Specifications 3.11.1.2, 3.11.2.2, 3.11.2.3 and 3.11.4.
- i. Degradation of containment structure, Specification 4.6.1.6.4.
- j. Steam Generator Tube Inspection, Specification 4.4.5.1.5.
- k. Accident Monitoring Instrumentation, Specification 3.3.3.8.
- l. Radiation Monitoring Instrumentation, Specification 3.3.3.1.
- m. Reactor Coolant System Vents, Specification 3.4.11.

## 6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE EVENTS.
- d. Records of surveillance activities, inspections, and calibrations required by these technical specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to operating procedures.

Docket No. 50-336  
B14946

Attachment 2

Millstone Nuclear Power Station, Unit No. 2

Proposed Revision to License Condition and  
Technical Specifications  
Fire Protection

Retyped Pages

December 1994



- (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitation for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended:
- (3) Pursuant to the Act and 10CFR Parts 30, 40, 70, to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required:
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensees are authorized to operate the facility at steady-state reactor core power levels not in excess of 2700 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 180, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Fire Protection

The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report and as approved in the SER dated September 19, 1978 and supplements dated October 21, 1980, November 11, 1981, October 31, 1985, April 15, 1986, January 15, 1987, April 29, 1988, and July 17, 1990 subject to the following provisions.

Amendment No.



The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(4) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training, and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 CFR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Millstone Nuclear Power Station Physical Security Plan," with revisions submitted through March 29, 1988; "Millstone Nuclear Power Station Suitability, Training, and Qualification Plan," with revision submitted through July 21, 1986; and "Millstone Nuclear Power Station Safeguards Contingency Plan," with revisions submitted through October 30, 1985. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

- D. This amended license is effective as of its date of issuance and shall expire at midnight July 31, 2015.

FOR THE NUCLEAR ENERGY COMMISSION

Roger S. Boyd, Acting Director  
Division of Reactor Licensing  
Office of Nuclear Reactor Regulation

Original Signed by Roger S. Boyd

Enclosures:

1. Incomplete Preoperational Test  
Items Which Must be Completed
2. Change No. 4 to Technical  
Specifications Contained in  
Appendix A to DPR-65

Date of Issuance: September 26, 1975

## INDEX

### LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.2 POWER DISTRIBUTION LIMITS</u>	
3/4.2.1 LINEAR HEAT RATE . . . . .	3/4 2-1
3/4.2.2 Deleted	
3/4.2.3 TOTAL INTEGRATED RADIAL PEAKING FACTOR - $F_r^T$ . . . . .	3/4 2-9
3/4.2.4 AZIMUTHAL POWER TILT . . . . .	3/4 2-10
3/4.2.5 Deleted	
3/4.2.6 DNB MARGIN . . . . .	3/4 2-13
<u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 REACTOR PROTECTIVE INSTRUMENTATION . . . . .	3/4 3-1
3/4.3.2 ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION . . . . .	3/4 3-10
3/4.3.3 MONITORING INSTRUMENTATION . . . . .	3/4 3-26
Radiation Monitoring . . . . .	3/4 3-26
Incore Detectors . . . . .	3/4 3-30
Seismic Instrumentation . . . . .	3/4 3-32
Meteorological Instrumentation . . . . .	3/4 3-36
Chlorine Detection Systems . . . . .	3/4 3-42
Accident Monitoring . . . . .	3/4 3-46
Radioactive Liquid Effluent Monitoring Instrumentation .	3/4 3-50
Radioactive Gaseous Effluent Monitoring Instrumentation .	3/4 3-56
<u>3.4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1 COOLANT LOOPS AND COOLANT CIRCULATION . . . . .	3/4 4-1
Startup and Power Operation . . . . .	3/4 4-1
Hot Standby . . . . .	3/4 4-1a
Shutdown . . . . .	3/4 4-1b

## INDEX

### LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

---

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.7 PLANT SYSTEMS</u>	
3/4.7.1 TURBINE CYCLE . . . . .	3/4 7-1
Safety Valves . . . . .	3/4 7-1
Auxiliary Feedwater Pumps . . . . .	3/4 7-4
Condensate Storage Tank . . . . .	3/4 7-6
Activity . . . . .	3/4 7-7
Main Steam Line Isolation Valves . . . . .	3/4 7-9
3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION . . . . .	3/4 7-10
3/4.7.3 REACTOR BUILDING CLOSED COOLING WATER SYSTEM . . . . .	3/4 7-11
3/4.7.4 SERVICE WATER SYSTEM . . . . .	3/4 7-12
3/4.7.5 FLOOD LEVEL . . . . .	3/4 7-13
3/4.7.6 CONTROL ROOM EMERGENCY VENTILATION SYSTEM . . . . .	3/4 7-16
3/4.7.7 SEALED SOURCE CONTAMINATION . . . . .	3/4 7-19
3/4.7.8 SNUBBERS . . . . .	3/4 7-21
3/4.7.9 DELETED . . . . .	3/4 7-33
3/4.7.10 DELETED . . . . .	3/4 7-33
3/4.7.11 ULTIMATE HEAT SINK . . . . .	3/4 7-34
 <u>3/4.8 ELECTRICAL POWER SYSTEMS</u>	
3/4.8.1 A.C. SOURCES . . . . .	3/4 8-1
Operating . . . . .	3/4 8-1
Shutdown . . . . .	3/4 8-5
3/4.8.2 ONSITE POWER DISTRIBUTION SYSTEMS . . . . .	3/4 8-6
A.C. Distribution - Operating . . . . .	3/4 8-6
A.C. Distribution - Shutdown . . . . .	3/4 8-7
D.C. Distribution - Operating . . . . .	3/4 8-8
D.C. Distribution - Shutdown . . . . .	3/4 8-10

## INDEX

### BASES

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<u>SECTION</u>	<u>PAGE</u>
<u>3/4.7 PLANT SYSTEMS</u>	
3/4.7.1 TURBINE CYCLE . . . . .	B 3/4 7-1
3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION . . .	B 3/4 7-3
3/4.7.3 REACTOR BUILDING CLOSED COOLING WATER SYSTEM . . . . .	B 3/4 7-3
3/4.7.4 SERVICE WATER SYSTEM . . . . .	B 3/4 7-4
3/4.7.5 FLOOD LEVEL . . . . .	B 3/4 7-4
3/4.7.6 CONTROL ROOM EMERGENCY VENTILATION SYSTEM . . . . .	B 3/4 7-4
3/4.7.7 SEALED SOURCE CONTAMINATION . . . . .	B 3/4 7-5
3/4.7.8 SNUBBERS . . . . .	B 3/4 7-5
3/4.7.9 DELETED . . . . .	B 3/4 7-6
3/4.7.10 DELETED . . . . .	B 3/4 7-7
3/4.7.11 ULTIMATE HEAT SINK . . . . .	B 3/4 7-7
 <u>3/4.8 ELECTRICAL POWER SYSTEMS</u> . . . . .	
 <u>3/4.9 REFUELING OPERATIONS</u>	
3/4.9.1 BORON CONCENTRATION . . . . .	B 3/4 9-1
3/4.9.2 INSTRUMENTATION . . . . .	B 3/4 9-1
3/4.9.3 DECAY TIME . . . . .	B 3/4 9-1
3/4.9.4 CONTAINMENT PENETRATIONS . . . . .	B 3/4 9-1
3/4.9.5 COMMUNICATIONS . . . . .	B 3/4 9-1
3/4.9.6 CRANE OPERABILITY--CONTAINMENT BUILDING . . . . .	B 3/4 9-2
3/4.9.7 CRANE TRAVEL--SPENT FUEL STORAGE BUILDING . . . . .	B 3/4 9-2
3/4.9.8 SHUTDOWN COOLING AND COOLING RECIRCULATION . . . . .	B 3/4 9-2

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## PLANT SYSTEMS

### 3/4.7.11 ULTIMATE HEAT SINK

#### LIMITING CONDITION FOR OPERATION

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3.7.11 The ultimate heat sink shall be OPERABLE with an average water temperature of less than or equal to 75°F at the Unit 2 intake structure.

APPLICABILITY: MODES 1, 2, 3, AND 4

#### ACTION:

With the requirements of the above specification not satisfied, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

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4.7.11 The ultimate heat sink shall be determined OPERABLE:

- a. At least once per 24 hours by verifying the average water temperature at the Unit 2 intake structure to be within limits.
- b. At least once per 6 hours by verifying the average water temperature at the Unit 2 intake structure to be within limits when the average water temperature exceeds 70°F.

## INSTRUMENTATION

### BASES

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3/4.3.3.6 DELETED

3/4.3.3.7 DELETED

3/4.3.3.8 Accident Monitoring Instrumentation

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. This capability is consistent with the recommendations of NUREG-0578, "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations".



## PLANT SYSTEMS

### BASES

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When the cause of the rejection of a snubber is clearly established and remedied for that snubber and for any other snubbers that may be generically susceptible, that snubber may be exempted from being counted as inoperable. Generically susceptible snubbers are those which are of a specific make or model and have the same design features directly related to rejection of the snubber by visual inspection, or are similarly located or exposed to the same environmental conditions such as temperature, radiation, and vibration. Due to the size and location of the steam generator hydraulic snubbers, regular removal and testing as specified for hydraulic and mechanical snubbers would represent a significant undertaking during each refueling outage. As such, these snubbers have been treated separately and are tested and refurbished as a group in accordance with the manufacturer's recommended preventative maintenance program.

When a snubber is found inoperable, an engineering evaluation is performed, in addition to the determination of the snubber mode of failure, in order to determine if any safety-related component or system has been adversely affected by the inoperability of the snubber.

The engineering evaluation shall determine whether or not the snubber mode of failure has imparted a significant effect or degradation on the supported component or system.

To provide assurance of snubber reliability, a representative sample of the installed snubbers will be tested during plant shutdowns at eighteen (18) month intervals. Observed failures of these sample snubbers shall require testing of additional units.

Hydraulic snubbers and mechanical snubbers may each be treated as a different entity for the above surveillance programs.

The service life of a snubber is evaluated via manufacturer input and information through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubber, seal replaced, spring replaced, in high radiation area, in high temperature area, etc....). The requirement to monitor the snubber service life is included to ensure that the snubbers periodically undergo a performance evaluation in view of their age and operating conditions. These records will provide statistical bases for future consideration of snubber service life. The requirements for the maintenance of records and the snubber service life review are not intended to affect plant operation.

3/4.7.9 DELETED

## PLANT SYSTEMS

### BASES

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#### 3/4.7.10 DELETED

#### 3/4.7.11 ULTIMATE HEAT SINK

The limitations on the ultimate heat sink temperature ensure that sufficient cooling capacity is available to either,

- 1) provide normal cooldown of the facility, or 2) to mitigate the effects of accident conditions within acceptable limits.

The limitations on maximum temperature are based on a 30-day cooling water supply to safety related equipment without exceeding their design basis temperature.

## ADMINISTRATIVE CONTROLS

### FACILITY STAFF (CONTINUED)

- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor. (Table 6.2-1)
- e. ALL CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions. These procedures should follow the general guidance of the NRC Policy Statement on working hours (Generic Letter No. 82-12).

### 6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for:
- a. If the Operations Manager does not hold a senior reactor operator license for Millstone Unit No. 2, then the Operations Manager shall have held a senior reactor operator license at a Pressurized Water Reactor other than Millstone Unit No. 2 and an individual serving in the capacity of the Assistant Operations Manager shall hold a senior reactor operator license for Millstone Unit No. 2.
  - b. The Shift Technical Advisor (STA) who shall meet the requirements of Specification 6.3.1.b.1 or 6.3.1.b.2.
    - 1. Dual-role individual: Must hold a senior reactor operator's license at Millstone Unit No. 2, meet the STA training criteria of NUREG-0737, Item I.A.1.1, and meet one of the following educational alternatives:
      - a. Bachelor's degree in engineering from an accredited institution;
      - b. Professional Engineer's license obtained by the successful completion of the PE examination;

TABLE 6.2-1<sup>(3)</sup>  
MINIMUM SHIFT-CREW COMPOSITION<sup>(2)</sup>

LICENSE CATEGORY	<u>APPLICABLE MODES</u>	
	1, 2, 3 & 4	5 & 6
Senior Reactor Operator	2	1 <sup>(1)</sup>
Reactor Operator	2	1
Non-Licensed Operator	2	1
Shift Technical Advisor	1 <sup>(4)</sup>	None Required

- (1) Does not include the licensed Senior Reactor or Senior Reactor Operator Limited to Fuel Handling individual supervision CORE ALTERATIONS after the initial fuel loading.
- (2) The above shift crew composition and the qualified health physics technician of Section 6.2.2 may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence provided expeditious actions are taken to fill the required position.
- (3) Requirements for minimum number of licensed operators on shift during operation in modes other than cold shutdown or refueling are contained in 10CFR50.54(m).
- (4) The Shift Technical Advisor position can be filled by either of the two Senior Reactor Operators (a dual-role individual), if he meets the requirements of Specification 6.3.1.b.1.

## ADMINISTRATIVE CONTROLS

### 6.4 TRAINING

A retraining and replacement training program for the facility staff shall be maintained under the direction of the Senior Vice President — Millstone Station and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.59. The Director-Nuclear Training has the overall responsibility for the implementation of the Training Program.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 Plant Operations Review Committee (PORC)

##### Function

- 6.5.1.1 The PORC shall function to advise the Nuclear Unit Director on all matters related to nuclear safety.

##### Composition

- 6.5.1.2 The PORC shall be composed of the:

Chairperson:	Nuclear Unit Director
Vice Chairperson & Member:	Operations Manager
Member:	Maintenance Manager
Member:	Instrument and Controls Manager
Member:	Reactor Engineer
Member:	Radiation Protection Supervisor or Chemistry Supervisor
Member:	Engineering Manager
Member:	Staff Engineer

##### Alternates

- 6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairperson to serve on a temporary basis; however, no more than two alternates shall participate in PORC activities at any one time.



## ADMINISTRATIVE CONTROLS

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### Meeting Frequency

- 6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairperson.

### Quorum

- 6.5.1.5 A quorum of the PORC shall consist of the Chairperson, or Vice Chairperson, or Senior Vice President — Millstone Station and four members including alternates.

### Responsibilities

- 6.5.1.6 The PORC shall be responsible for:
- a. Review of 1) all procedures, except common site procedures, required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Nuclear Unit Director to affect nuclear safety.
  - b. Review of all proposed tests and experiments that affect nuclear safety.
  - c. Review of all proposed changes to Sections 1.0 - 5.0 of these Technical Specifications.
  - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
  - e. Investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Executive Vice President-Nuclear and to the Chairperson of the Nuclear Review Board.
  - f. Review of all REPORTABLE EVENTS.
  - g. Review of facility operations to detect potential safety hazards.
  - h. Performance of special reviews and investigations and reports thereon as requested by the Chairperson of the Nuclear Review Board.
  - i. Render determinations in writing if any item considered under 6.5.1.6(a) through (d) above, as appropriate and as provided by 10CFR50.59 or 10CFR50.92, constitutes an unreviewed safety question or requires a significant hazards consideration determination.
  - j. Review of the fire protection program and implementing procedure.

## ADMINISTRATIVE CONTROLS

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### Meeting Frequency

- 6.5.2.4 The SORC shall meet at least once per six months and as convened by the SORC Chairperson.

### Quorum

- 6.5.2.5 A quorum of the SORC shall consist of the Chairperson or Vice Chairperson and five members including alternates.

### Responsibilities

- 6.5.2.6 The SORC shall be responsible for:
- a. Review of 1) all common site procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Senior Vice President — Millstone Station to affect site nuclear safety.
  - b. Review of all proposed changes to Section 6.0 "Administrative Controls" of these Technical Specifications.
  - c. Performance of special reviews and investigations and reports as requested by the Chairperson of the Site Nuclear Review Board.
  - d. Review of the Plant Security Plan and implementing procedures and shall submit changes to the Chairperson of the Site Nuclear Review Board.
  - e. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairperson of the Site Nuclear Review Board.
  - f. Review of all common site proposed tests and experiments that affect nuclear safety.
  - g. Review of all common site proposed changes or modifications to systems or equipment that affect nuclear safety.
  - h. Render determinations in writing or meeting minutes if any item considered under 6.5.2.6(a) through (g) above, as appropriate and as provided by 10CFR50.59 or 10CFR50.92, constitutes an unreviewed safety question or requires a significant hazards consideration determination.
  - i. Review of the common site fire protection program and implementing procedures.

### Authority

- 6.5.2.7 The SORC shall:
- a. Recommend to the Senior Vice President— Millstone Station written approval or disapproval in meeting minutes of items considered under 6.5.2.6(a) through (g) above.

## ADMINISTRATIVE CONTROLS

### SPECIAL REPORTS (CONT.)

- b. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- c. Safety Class 1 Inservice Inspection Program Review, Specification 4.4.10.1.
- d. ECCS Actuation, Specifications 3.5.2 and 3.5.3.
- e. Deleted
- f. Deleted
- g. RCS Overpressure Mitigation, Specification 3.4.9.3.
- h. Radiological Effluent Reports required by Specifications 3.11.1.2, 3.11.2.2, 3.11.2.3 and 3.11.4.
- i. Degradation of containment structure, Specification 4.6.1.6.4.
- j. Steam Generator Tube Inspection, Specification 4.4.5.1.5.
- k. Accident Monitoring Instrumentation, Specification 3.3.3.8.
- l. Radiation Monitoring Instrumentation, Specification 3.3.3.1.
- m. Reactor Coolant System Vents, Specification 3.4.11.

### 6.10 RECORD RETENTION

- 6.10.1 The following records shall be retained for at least five years:
- a. Records and logs of facility operation covering time interval at each power level.
  - b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
  - c. ALL REPORTABLE EVENTS.
  - d. Records of surveillance activities, inspections, and calibrations required by these technical specifications.
  - e. Records of reactor tests and experiments.
  - f. Records of changes made to operating procedures.