



Nebraska Public Power District

COOPER NUCLEAR STATION
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NLS940088
December 19, 1994

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Gentlemen:

Subject: Updated Response to Station Blackout Rule 10CFR50.63
Cooper Nuclear Station
NRC Docket No. 50-298, DPR-46

- Reference:
- 1) Letter from G. A. Trevors to USNRC dated March 23, 1990, Supplemental Response to Station Blackout Rule.
 - 2) Letter from G. R. Horn to USNRC dated January 31, 1991, Station Blackout Information Request.
 - 3) Letter from G. R. Horn to USNRC dated September 30, 1991, Response to Recommendations on Station Blackout.
 - 4) Letter from USNRC to G. R. Horn dated August 22, 1991, CNS Station Blackout Safety Evaluation Report Technical Evaluation Report (SER/TER).

The purpose of this letter is to provide clarification to the District's responses in the above referenced transmittals concerning Station Blackout at Cooper Nuclear Station.

The District identified RCIC as a system which may be relied upon for automatic operation in its Station Blackout Coping Analysis. However, the District determined that RCIC-MO-MO14, the turbine trip/throttle valve, was AC powered. During a Station Blackout event, the RCIC system would have initiated; however, it would not have been capable of automatically resetting after a high water level trip of the system.

The District has modified the RCIC System by installation of Design Change 94-267 to change the RCIC-MO-MO14 to DC powered. This modification will provide for automatic reset of RCIC after a high water level trip during a station blackout event.

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In Reference 1, the District stated that no modifications were required for Station Blackout. The modification, DC 94-267, was required for the CNS response to Station Blackout 10CFR50.63.

In References 2 and 3, the District stated that Reactor Vessel level is controlled by automatic operation of RCIC. Design Change 94-267 was required to provide automatic operation of RCIC after the first cycle during a Station Blackout event.

In reference 4, the SER/TER stated that no hardware modifications were required at CNS to attain the proposed coping duration of 4 hours. As explained previously, DC 94-267 was required to provide automatic operation of RCIC.

Should you have any questions concerning these clarifications or require additional information, please contact me.


John H. Mueller
Site Manager

/nr

cc: Regional Administrator
USNRC - Region IV
Arlington, Texas

NRC Resident Inspector
Cooper Nuclear Station

NPG Distribution

Correspondence No: NLS-9088

The following table identifies those actions committed to by the District in this document. Any other actions discussed in the submittal represent intended or planned actions by the District. They are described to the NRC for the NRC's information and are not regulatory commitments. Please notify the Licensing Manager at Cooper Nuclear Station of any questions regarding this document or any associated regulatory commitments.

COMMITMENT	COMMITTED DATE OR OUTAGE
None	