



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038  
Hope Creek Generating Station

February 15, 1995

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT  
HOPE CREEK GENERATION STATION UNIT 1  
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for January are being forwarded to you with the summary of changes, tests, and experiments that were implemented during January 1995 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

*[Signature]* for RSH  
R. J. Hovey  
General Manager -  
Hope Creek Operations

*[Signature]*  
DR:WS:JC  
Attachments

C Distribution

250080

The Enormous Doodle

9502220137 950131  
PDR ADDCK 05000354  
R PDR

*[Handwritten initials]*

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# OPERATING DATA REPORT

DOCKET NO. 50-354  
 UNIT Hope Creek  
 DATE 02/10/95  
 COMPLETED BY D. W. Lyons  
 TELEPHONE (609) 339-3517

## OPERATING STATUS

1. Reporting Period January 1995 Gross Hours in Report Period 744

2. Currently Authorized Power Level (MWt) 3293  
 Max. Depend. Capacity (MWe-Net) 1031  
 Design Electrical Rating (MWe-Net) 1067

3. Power Level to which restricted (if any) (MWe-Net) None

4. Reasons for restriction (if any)

	This Month	Yr To Date	Cumulative
5. No. of hours reactor was critical	<u>744.0</u>	<u>744.0</u>	<u>60679.9</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
7. Hours generator on line	<u>744.0</u>	<u>744.0</u>	<u>59747.4</u>
8. Unit reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. Gross thermal energy generated (MWH)	<u>2410230</u>	<u>2410230</u>	<u>190824575</u>
10. Gross electrical energy generated (MWH)	<u>811347</u>	<u>811347</u>	<u>63239013</u>
11. Net electrical energy generated (MWH)	<u>778188</u>	<u>778188</u>	<u>60431504</u>
12. Reactor service factor	<u>100.0</u>	<u>100.0</u>	<u>85.3</u>
13. Reactor availability factor	<u>100.0</u>	<u>100.0</u>	<u>85.3</u>
14. Unit service factor	<u>100.0</u>	<u>100.0</u>	<u>84.0</u>
15. Unit availability factor	<u>100.0</u>	<u>100.0</u>	<u>84.0</u>
16. Unit capacity factor (using MDC)	<u>101.5</u>	<u>101.5</u>	<u>82.4</u>
17. Unit capacity factor (Using Design MWe)	<u>98.0</u>	<u>98.0</u>	<u>79.6</u>
18. Unit forced outage rate	<u>0.0</u>	<u>0.0</u>	<u>4.7</u>
19. Shutdowns scheduled over next 6 months (type, date, & duration):			
None			
20. If shutdown at end of report period, estimated date of start-up:			
N/A			

OPERATING DATA REPORT  
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-354  
UNIT Hope Creek  
DATE 02/10/95  
COMPLETED BY D. W. Lyons  
TELEPHONE (609) 339-3517

MONTH January 1995

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENTS
1.	1/14	S	0	H	5 - POWER REDUCTION >20%.	UNIT POWER WAS REDUCED TO PERFORM - ROD SWAPS - SCRAM TIMING - HCU MAINTENANCE - CIV TEST

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354  
UNIT Hope Creek  
DATE 02/10/95  
COMPLETED BY D. W. Lyons  
TELEPHONE (609) 339-3517

MONTH January 1995

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	<u>1052</u>
2.	<u>1060</u>
3.	<u>1056</u>
4.	<u>1071</u>
5.	<u>1064</u>
6.	<u>1060</u>
7.	<u>1060</u>
8.	<u>1052</u>
9.	<u>1063</u>
10.	<u>1061</u>
11.	<u>1059</u>
12.	<u>1056</u>
13.	<u>1052</u>
14.	<u>801</u>
15.	<u>1044</u>
16.	<u>1051</u>

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	<u>1060</u>
18.	<u>1058</u>
19.	<u>1061</u>
20.	<u>1058</u>
21.	<u>915</u>
22.	<u>1051</u>
23.	<u>1075</u>
24.	<u>1061</u>
25.	<u>1054</u>
26.	<u>1074</u>
27.	<u>1064</u>
28.	<u>1065</u>
29.	<u>1059</u>
30.	<u>1062</u>
31.	<u>1032</u>

# REFUELING INFORMATION

DOCKET NO. 50-354  
UNIT Hope Creek 1  
DATE 02/10/95  
COMPLETED BY R. Schmidt  
TELEPHONE (609) 339-3740

MONTH January 1995

1. Refueling information has changed from last month:  
Yes                      No X
2. Scheduled date for next refueling: 10/14/95
3. Scheduled date for restart following refueling: 11/13/95
4. A. Will Technical Specification changes or other license amendments be required?  
Yes                      No X
- B. Has the Safety Evaluation covering the COLR been reviewed by the Station Operating Review Committee?  
Yes                      No X  
If no, when is it scheduled? August 28, 1995
5. Scheduled date(s) for submitting proposed licensing action:  
Not required.
6. Important licensing considerations associated with refueling:  
N/A
7. Number of Fuel Assemblies:  

A. Incore	<u>764</u>
B. In Spent Fuel Storage (prior to refueling)	<u>1240</u>
C. In Spent Fuel Storage (after refueling)	<u>1472</u>
8. Present licensed spent fuel storage capacity: 4006  
Future spent fuel storage capacity: 4006
9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: 5/3/2006  
(EOC13)  
(Does allow for full-core offload)  
(Assumes 244 bundle reloads every 18 months until then)  
(Does not allow for smaller reloads due to improved fuel)

# HOPE CREEK GENERATING STATION

## MONTHLY OPERATING SUMMARY

January 1995

Hope Creek entered the month of January operating at 100% power. From 0200 hours on January 14 until 0500 hours on January 15, 1995, power was reduced to perform control rod swaps, scram timing, hydraulic control unit maintenance and the weekly CIV Test. On January 21 from 0200 hours until 2359 hours, unit power was reduced to complete the control rod swaps and do the weekly CIV Test. On January 31, 1995, power was reduced to 95% because of a transient in the "C" Feedwater Heater Train. There were no other major power reductions or scrams. As of January 31 1995, the unit has been on line for 112 consecutive days.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS  
FOR THE HOPE CREEK GENERATING STATION

January 1995

The following items have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.



## Temporary Modification

## Summary of Safety Evaluation

T-Mod 95-02: This Temporary Modification lifts leads on the signal resistor unit (SRU) that feeds the cooling tower average basin water temperature loop signal summator. These instruments feed CRIDS point (A2769) which is used by the shift to determine when to enter and exit the cooling tower de-icing fill bypass modes. This data summator averages four Temperature Elements (TE's) inside the cooling tower basin. One of the TE's has a failed temperature transmitter which is sending an erroneous (down scale) signal to the summator affecting the average temperature read at A2769. This T-Mod isolates the "B" input at the SRU and combines the output with that of the "A" to provide a 50% weighted average 4 input summer. A new transmitter has been ordered and is expected to be received by 3/31/95.

Failure of the entire Circulating Water System does not compromise any safety related system or component or prevent a safe shutdown of the plant. The only postulated accident evaluated in the FSAR involves the complete rupture of one of the expansions joints which is located in the turbine building. The SAR states that this does not adversely affect any safety related system. The complete failure of the basin temperature averaging circuit can not have any affect on the expansion joint failure described in the FSAR Section 10.4.5.3.

Therefore, this Temporary Modification does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.

T-Mod 95-004: This Temporary Modification installs steam heat tracing on the Station Auxiliary Boilers for the purposes of preventing instrument line freeze-up as climate conditions cause outside temperatures to fall below 32°F.

Failure of the installed tubing for the heat tracing presents no credible failure mode associated with this change. The auxiliary boiler steam system has no safety related function. The system is designed so that the failure of the system or a system component does not compromise any safety related systems or components or prevents a safe reactor shutdown.

Therefore, this Temporary Modification does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.

Other      Summary of Safety Evaluation

UFSAR Section 2.3 Meteorology Change: This UFSAR change includes the removal of detailed information provided in Table 2.3-29 and text reference to this information contained on page 2.3-36 of the UFSAR. Specifically the change involves removal of meteorological instrumentation and strip chart manufacturer, model specifics from the UFSAR. This change will make this section more consistent with other sections of the HC UFSAR which do not include this level of detail. There is no analysis of accidents evaluated in the HC UFSAR which include the Artificial Island Monitoring Program or meteorological instrumentation in the evaluation.

Therefore, this UFSAR change does not increase the probability or consequences of an accident previously described in the SAR and does not involve an Unreviewed Safety Question.

Revision of Previous Report:

A previous report submitted to the Commission dated November 15, 1994 reported that a station operating procedure HC.OP-SO.AB-001(Q) would be revised and required a 50.59 safety evaluation. A 50.59 was prepared as a contingency action in the event the procedure revision was needed however, the procedure revision was not required or implemented.