



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

February 15, 1995

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Evaluation of LOCA + SSE Load Effects on Steam Generator Tubes

Byron Nuclear Power Station, Unit 1
NPF-37, NRC Docket No. 50-454

Braidwood Nuclear Power Station, Unit 1
NPF-72, NRC Docket No. 50-456

- References:
1. George F. Dick, Jr. letter to D. L. Farrar transmitting the Technical Specification and associated Safety Evaluation Implementing an Interim Plugging Criteria for Byron Unit 1 Steam Generator Tubes, dated October 24, 1994
 2. Denise M. Saccomando letter to the NRC Document Control Desk transmitting a Schedule for LOCA+SSE Steam Generator Load Evaluation for Byron Station Unit 1 and Braidwood Station Unit 1, dated November 23, 1994

In Reference (1), the Nuclear Regulatory Commission (NRC) granted approval for the application of a voltage-based Tube Support Plate (TSP) Interim Plugging Criteria (IPC) to disposition steam generator (SG) tubes for continued service that are experiencing outside diameter stress corrosion cracking (ODSCC) confined within the thickness of the TSPs for Byron Unit 1, Cycle 7. Reference (1) also contained a requirement for the Commonwealth Edison Company (ComEd) to submit a schedule for evaluating the effects of the loads from a postulated loss-of-coolant accident (LOCA) in combination with a postulated safe shutdown earthquake (SSE) on certain tubes in the vicinity of the TSP wedges that react with the tube bundle load in the Byron Unit 1 SGs. In Reference (2), ComEd committed to provide that evaluation to the NRC by February 17, 1995. ComEd also stated that due to the similarities between Byron Unit 1 and Braidwood Unit 1, the evaluation would also be applicable to Braidwood Unit 1.

The evaluation of the effects of combined LOCA + SSE loads on certain tubes in the vicinity of the TSP wedges that react with the tube bundle load in the Byron Unit 1 and Braidwood Unit 1 SGs is contained in WCAP-14046, "Braidwood Unit 1 Technical Support for Cycle 5 Steam Generator Interim Plugging Criteria," Revision 2 (Proprietary). This revision modifies only Section 4.7 of WCAP-14046, Revision 1. Since no other sections of WCAP-14046, Revision 1 were modified as a result of this evaluation, they are not included in this revision.

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Accordingly, please find enclosed one copy each of Westinghouse authorization letter, CAW-95-783, accompanying affidavit, Proprietary Information Notice, Copyright Notice, WCAP-14046, "Braidwood Unit 1 Technical Support for Cycle 5 Steam Generator Interim Plugging Criteria," Revision 2 (Proprietary), and WCAP-14047, "Braidwood Unit 1 Technical Support for Cycle 5 Steam Generator Interim Plugging Criteria," Revision 2 (Non-Proprietary).

As WCAP-14046, Revision 2 contains information proprietary to Westinghouse Electric Corporation, it is supported by an affidavit signed by Westinghouse, the owner of the information. The affidavit sets forth the basis on which the information may be withheld from public disclosure by the Commission and addresses with specificity the considerations listed in paragraph (b)(4) of Section 2.790 of the Commission's regulations. Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure in accordance with 10 CFR 2.790 of the Commission's regulations.

Correspondence with respect to the proprietary aspects of the items listed above or the supporting Westinghouse Affidavit should reference CAW-95-783 and should be addressed to N. J. Liparulo, Manager of Nuclear Safety & Regulatory Activities, Westinghouse Electric Corporation, P. O. Box 355, Pittsburgh, Pennsylvania 15230-0355.

Please address any comments or questions regarding this matter to this office.

Sincerely,



Harold D. Pontious, Jr.
Nuclear Licensing Administrator

- Enclosure 1: Westinghouse Letter CAW-95-783
- Enclosure 2: WCAP-14046, Revision 2 (Proprietary)
- Enclosure 3: WCAP-14047, Revision 2 (Non-Proprietary)

cc: J. B. Martin, Regional Administrator - RIII
M. D. Lynch, Senior Project Manager - NRR
R. R. Assa, Braidwood Project Manager - NRR
G. F. Dick Jr., Byron Project Manager - NRR
S. G. DuPont, Senior Resident Inspector - Braidwood
H. Peterson, Senior Resident Inspector - Byron
Office of Nuclear Facility Safety - IDNS