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February 9, 1995
C311-95-2077

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for January 1995

Enclosed are two copies of the January 1995 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

T. G. Broughton
Vice President and Director, TMI

WGH

Attachments

cc: Administrator, Region I
TMI Senior Resident Inspector
T95001

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OPERATIONS SUMMARY

January 1995

The plant entered the month operating at 100% power and remained at that level throughout the entire month. Net unit electrical output averaged approximately 822 MWe during January.

MAJOR SAFETY RELATED MAINTENANCE

The following is a summary of major safety related maintenance items accomplished during the month.

NUCLEAR SERVICE RIVER WATER PIPING

A pin hole leak was discovered in a section of 12" piping downstream of NS-C-1B backwash valve NR-V-12B during December. Corrective maintenance performed in January involved welding a threaded half coupling over the defect and installing a threaded pipe plug in the half coupling. All post maintenance weld inspections and leak test results were satisfactory.

EMERGENCY DIESEL ENGINE EG-Y-1B

Emergency Diesel EG-Y-1B was removed from service in January because of a fuel oil leak on the #5 injector pump on the control side of the engine. A new injector was installed and the engine tested satisfactory with no leakage evident.

RIVER WATER HEAT EXCHANGER INSPECTION

The River Water Heat Exchanger inspection program continued into January. Three Heat Exchangers were inspected. Secondary Service Closed Cooler, SC-C-1A, was leak tested with no leaking tubes identified. Eddy Current Testing of 80 tubes revealed no deficiencies that required tube plugging. One tube was removed from the cooler for further inspection. Its location in the tubesheet was plugged.

Leak testing of Secondary Service Closed Cooler, SC-C-1B, identified one leaking tube. Eddy Current Testing of 235 tubes identified 12 tubes with defects. The 13 tubes were stabilized and plugged.

Nuclear Service Closed Cooler, NS-C-1D, was leak checked with no leaking tubes identified. Eddy Current Testing of all 800 tubes revealed deficiencies in 10 tubes that required stabilizers and plugging.

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE February 9, 1995
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

OPERATING STATUS

1. UNIT NAME: THREE MILE ISLAND UNIT 1
2. REPORTING PERIOD: JANUARY 1995
3. LICENSED THERMAL POWER: 2568
4. NAMEPLATE RATING (GROSS MWe): 871
5. DESIGN ELECTRICAL RATING (NET MWe): 819
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
		-----	-----	-----
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	744.0	178969.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	744.0	102332.7
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	744.0	101199.1
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1908743	1908743	247825014
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	647188	647188	83330442
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	611412	611412	78266830
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	56.5
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	56.5
21. UNIT CAPACITY FACTOR (USING MDC NET)		104.6	104.6	55.6
22. UNIT CAPACITY FACTOR (USING DER NET)		100.3	100.3	53.4
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	37.4
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60761.2
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE February 9, 1995
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

MONTH: JANUARY

DAY AVERAGE DAILY POWER LEVEL
 (MWe-NET)

1	821
2	823
3	824
4	825
5	823
6	821
7	823
8	824
9	823
10	825
11	823
12	822
13	821
14	811
15	806
16	815

DAY AVERAGE DAILY POWER LEVEL
 (MWe-NET)

17	820
18	820
19	820
20	818
21	822
22	824
23	825
24	824
25	823
26	824
27	824
28	826
29	826
30	825
31	824

DOCKET NO. 50-289

UNIT NAME TMI-1

DATE February 9, 1995

COMPLETED BY W. G. Heysek

TELEPHONE (717) 948-8191

REPORT MONTH January 1995

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	License Event Report#	System Code ' & "	Component Code ' & "	Cause & Corrective Action to Prevent Recurrence
						None			

1. F Forced
S Scheduled
2. Reason
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)
3. Method
1-Manual
2-Manual Scan
3-Automatic Scan
4-Other (Explain)
4. Exhibit G - Instructions for
preparation of Data Entry Sheets
for License Event Report (LER)
File (NUREG-24761)
5. Exhibit 1 same source
6. Actually used exhibits F & H NUREG 0141

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **September 8, 1995**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **NO**
5. Scheduled date(s) for submitting proposed licensing action and supporting information: **NA**
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
 - a) TMI will use the new Mark B10 fuel assembly in the Cycle 11 reload batch which is an upgraded design of the Mark B9 assembly used in Cycle 10. The Mark B10 provides a leaf-type cruciform assembly hold-down spring to replace the previous coil spring design which has experienced random failures during operation and requires visual inspection each outage. The Mark B10 design meets all current BWFC fuel design criteria and is in use at other B&W 177 FA plants.
 - b) TMI also will use four new Westinghouse Lead Test Assemblies (LTA) in the Cycle 11 reload batch. Their planned operation is for three consecutive cycles with discharge at end-of-Cycle 13.

The four W LTAs inserted in Cycle 9 were discharged at EOC-9 due to detection of fuel rod failures caused by grid-to-rod fretting similar to that seen in W Vantage 5H fuel designs. The Cycle 11 LTAs will use the generic W recommended design fix of rotated intermediate spacer grids to minimize flow-induced fuel vibrations and thus eliminate fretting. A prototype LTA will be flow-tested to demonstrate the effectiveness of the fix. The production LTA will use ZIRLO fuel rod cladding, guide tubes and instrumentation tube and intermediate grids in place of Zircaloy 4 materials used for the Cycle 9 LTAs. Otherwise, the Cycle 11 LTA design is basically the same as the Cycle 9 design.

The LTAs will meet current W fuel design criteria while operating within TMI core operating limits. LTA enrichment and core location will ensure that an LTA will not be the lead (hot) assembly at any time during the cycle and will not set any safety or operating limits. The LTAs will remain bounded by existing UFSAR safety analyses results.

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 601

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase I of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through the end of the current operating license and beyond.