



Boston Edison

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier

Vice President Nuclear Operations
and Station Director

February 14, 1995
BECO Ltr. #95-021

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPK-35

JANUARY 1995 MONTHLY REPORT

In accordance with PNPS Technical Specification 6.9 A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.


L.J. Olivier

WJM/lam/9458

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

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OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE _____
 COMPLETED BY: W. Munro
 TELEPHONE (508) 830-8474

OPERATING STATUS

NOTES

- | | | |
|----|---|--------------|
| 1. | Unit Name | Pilgrim I |
| 2. | Reporting Period | January 1995 |
| 3. | Licensed Thermal Power (MWt) | <u>1998</u> |
| 4. | Nameplate Rating (Gross MWe) | <u>678</u> |
| 5. | Design Electrical Rating (Net MWe) | <u>655</u> |
| 6. | Maximum Dependable Capacity (Gross MWe) | <u>696</u> |
| 7. | Maximum Dependable Capacity (Net MWe) | <u>670</u> |
| 8. | If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons: | |

NONE

- | | |
|-----|--|
| 9. | Power Level To Which Restricted, If Any (Net MWe): <u>None</u> |
| 10. | Reasons For Restrictions, If Any: <u>N/A</u> |

	<u>This Month</u>	<u>Yr-to-Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>744.0</u>	<u>744.0</u>	<u>194136.0</u>
12. Hours Reactor Critical	<u>744.0</u>	<u>744.0</u>	<u>119974.1</u>
13. Hours Reactor Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>744.0</u>	<u>115605.1</u>
15. Hours Unit Reserve Shutdown	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated(MWH)	<u>1485600.0</u>	<u>1485600.0</u>	<u>204240816.0</u>
17. Gross Electrical Energy Generated(MWH)	<u>513420.0</u>	<u>513420.0</u>	<u>69133094.0</u>
18. Net Electrical Energy Generated(MWH)	<u>494219.0</u>	<u>494219.0</u>	<u>66451996.0</u>
19. Unit Service Factor	<u>100.0</u>	<u>100.0</u>	<u>59.6</u>
20. Unit Availability Factor	<u>100.0</u>	<u>100.0</u>	<u>59.6</u>
21. Unit Capacity Factor (Using MDC Net)	<u>99.1</u>	<u>99.1</u>	<u>51.1</u>
22. Unit Capacity Factor (Using DER Net)	<u>101.4</u>	<u>101.4</u>	<u>52.3</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>12.4</u>
24. Shutdowns Scheduled Over Next 6 Months			
	(Type, Date, and Duration of Each) RFO-10, March 25, 1995 for 55 days		
25. If Shutdown at End of Report Period,			
	Estimated Date of Startup - Unit Operating		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-293
DATE: _____
COMPLETED BY: W. Munro
TELEPHONE: (508) 830-3474

MONTH January 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	664	17	664
2	664	18	663
3	665	19	664
4	664	20	664
5	665	21	664
6	664	22	664
7	665	23	664
8	665	24	664
9	665	25	664
10	665	26	664
11	665	27	664
12	665	28	664
13	665	29	663
14	664	30	664
15	664	31	665
16	664		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR JANUARY 1995

The unit started the reporting period at approximately 100 percent Core Thermal power and essentially maintained this level for the entire reporting period.

SAFETY RELIEF VALVE CHALLENGES

MONTH OF JANUARY 1995

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: March 25, 1995.
3. Scheduled date for restart following next refueling: May 19, 1995.
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1993 refueling outage was of the same design as loaded in the previous refueling outage and consisted of 140 assemblies.
7.
 - (a) There are 580 fuel assemblies in the core.
 - (b) There are 1629 fuel assemblies in the spent fuel pool.
8.
 - (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2878 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.
 - (b) The planned spent fuel storage capacity is 3859 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1226 fuel assemblies.

MONTH JANUARY 1995

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Primary Containment System*	Differential Pressure Transmitter (PTD-5021)	During performance of Procedure 8.M.2-6.2 primary containment pressure boundary was degraded due to an uninstalled plug on Differential Pressure Transmitter PTD-5021	Root cause was utility non-licensed I&C Technician, Supervisor and Nuclear Watch Engineer (NWE) error. Inattention to detail by the I&C Technician who performed and the Supervisor and NWE who reviewed the previous calibration of the transmitter on November 22, 1994	Installation of a plug for the Differential Pressure Transmitter PTD-5021	Maintenance personnel received directions from the Maintenance Manager the Plant manager and the Site Director regarding the requirements for procedure signoffs and verifications. The NWE received directions from the Operations Section Manager, Plant Manager, and the Site Director regarding his responsibilities for sign-off of procedures. Refer to associated LER for additional details	94-007-00
Reactor Core Isolation Cooling (RCIC) System	Oil temperature switch TS-1300-10	Broken wire discovered during system walkdown	Undetermined	Broken wire replaced	To be determined	N/A
Containment Vacuum Relief System	Solenoid valve SV-5040B fuse.	Actuation of the reactor-to-torus train "B" Vacuum Relief System while operating at 100 percent power	Loose fuse in the control circuit of Solenoid Valve SV-5040B	Fuse clip tension was increased and new fuse installed	Refer to associated LER	95-001-00

*Event was discovered December 28, 1994; LER sent out 1/27/95

UNIT SHUTDOWNS AND POWER REDUCTIONS DOCKET NO: 50-293DOCKET NO: 50-293NAME: Pilgrim I

DATE: _____

COMPLETED BY: W. MunroTELEPHONE: (508) 830-8474REPORT MONTH: January 1995

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONEN CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
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There were no unit shutdowns or significant power reductions to report during the reporting period.

1	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Operator Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)